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The Monte Carlo Method and MCNP – A Brief Review of Our 40 Year History

Presentation to the International Topical Meeting on Industrial Radiation and Radioisotope Measurement Applications Conference

Avneet Sood, PhD

XCP-3 Group Leader

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LA-UR-17-25633

Outline

- **Origins of the Monte Carlo method**
  - Development of electronic computers and Monte Carlo method occur simultaneously
  - Ulam, Von Neumann, Richtmeyer, Metropolis, Fermi

- **Growth and usage of Monte Carlo codes**
  - 1950's, 1960's, and 1970's
    - Monte Carlo becomes mainstream; nuclear criticality and reactor

- **Emergence of MCNP**

- **MCNP's history**

- **MCNP's upcoming future**

*nearly all references can be found at: https://laws.lanl.gov/vhosts/mcnp.lanl.gov/references.shtml
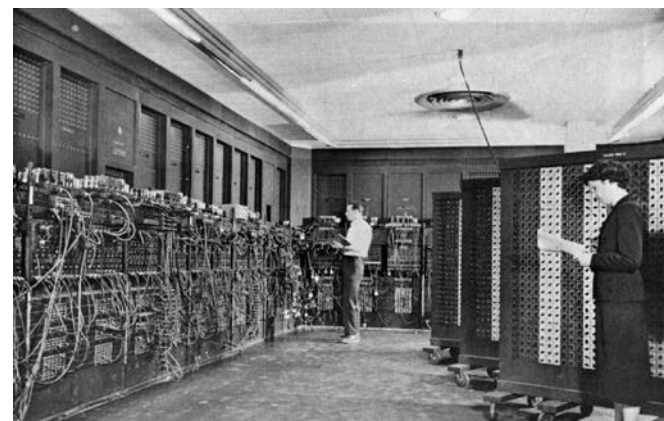
Abstract

The Monte Carlo method for radiation particle transport has its origins at LANL dating back to the 1940's. The creators of these methods were Drs. Stanislaw Ulam, John von Neumann, Robert Richtmyer, and Nicholas Metropolis. Monte Carlo methods for particle transport have been driving computational developments since the beginning of modern computers; this continues today. In the 1950's and 1960's, these new methods were organized into a series of special-purpose Monte Carlo codes, including MCS, MCN, MCP, and MCG. These codes were able to transport neutrons and photons for specialized LANL applications. In 1977, these separate codes were combined to create the first generalized Monte Carlo radiation particle transport code, MCNP. In 1983, MCNP3 was released for public distribution to the Radiation Safety Information Computational Center (RSICC). The upcoming release of MCNP (version 6.2) is expected in June 2017. Approximately 20,000 copies of MCNP have been distributed to users in government institutions, academia, and private industries worldwide. This talk will review our history, current status, and future directions.

The Origins of Monte Carlo – 1946 Stanislaw Ulam

- "*The year was 1945. Two earthshaking events took place: the successful test at Alamogordo and the building of the first electronic computer*" – N. Metropolis

- **The method was invented by Stanislaw Ulam in 1946 playing Solitaire while recovering from an illness.**

- "*After spending a lot of time trying to estimate success by combinatorial calculations, I wondered whether a more practical method…might be to lay it out say one hundred times and simply observe and count the number of successful plays*" – S. Ulam

Stanislaw Ulam

ENIAC– the first electronic computer, University of Pennsylvania. Solved ballistic trajectory problems for Army Ballistics Research Lab. Used electron tubes instead of mechanical counters. Minutes instead of days. Declassified in 1946.

Trinity – code name for first nuclear detonation

"Stan Ulam, John von Neumann, and the Monte Carlo Method," R. Eckhardt, Los Alamos Science Special Issue 1987.

The Origins of the Monte Carlo Method

- **Ulam describes this idea to John von Neumann in a conversation in 1946**

- **Von Neumann is intrigued**
  - 1943: Electro-Mechanical computers solved non-linear diff. eq. via production line. Punch card used for every point in space/time
  - New computers could count/arithmetic and hence solve difference equations (BRL at Aberdeen, MD)
  - Statistical sampling on electronic computers
  - Especially suitable for exploring neutron chain reactions in fission – neutron multiplication rates

- **R.D Richtmyer and J. von Neumann "Statistical Methods in Neutron Diffusion", Los Alamos (LAMS-557) April 9, 1947.**
  - Detailed letter from John von Neumann to Robert Richtmyer describing a conversation in March 1947
    - "I have been thinking a good deal about the possibility of using statistical methods to solve neutron diffusion and multiplication problems in accordance with the principle suggested by Stan Ulam"
    - Letter contained 81-step pseudo code for using MC for particle transport
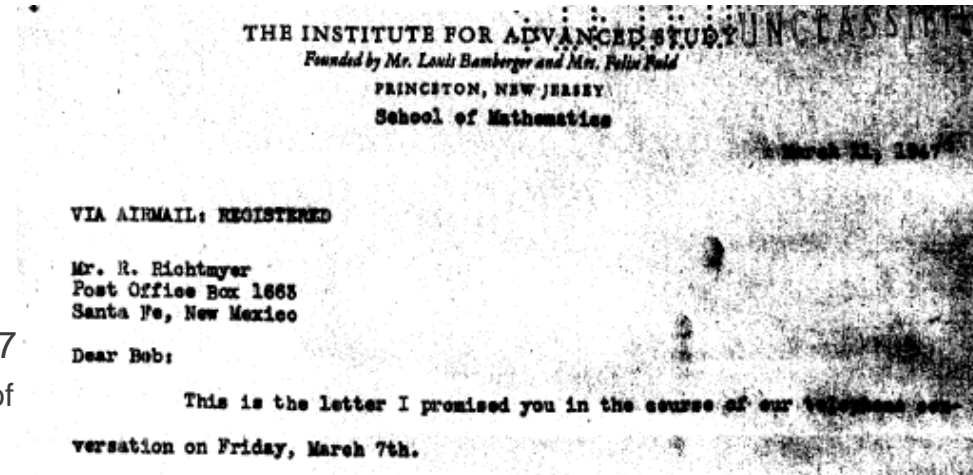

J. Von Neumann invented scientific computing in the 1940's
- Stored programs now called software
- Algorithms/Flowcharts
- Hardware design

John von Neumann

Consultant to Aberdeen and Los Alamos

THE INSTITUTE FOR ADVANCED STUDY UNCLASSIFIED
Founded by Mr. Louis Bamberger and Mrs. Felix Fuld
PRINCETON, NEW JERSEY
School of Mathematics

VIA AIRMAIL: REGISTERED

Mr. R. Richtmyer
Post Office Box 1663
Santa Fe, New Mexico

Dear Bob:

This is the letter I promised you in the course of our telephone conversation on Friday, March 7th.

The first Monte Carlo (pseudo) Code - 1947

- **Von Neumann's Assumptions:**
  - Time-dependent, continuous energy, spherical but radially-varying, 1 fissionable material, isotropic scattering and fission production, fission multiplicities of 2,3, or 4

- **Suggested 100 neutrons each to be run for 100 collisions**
  - Thought these were too much

- **Estimated time: 5 hrs on ENIAC**

- **Richtmyer's response:**
  - Very interested in idea and proposed suggestions
    - Allow for multiple fissionable materials, no fission spectrum energy dependence, single neutron multiplicity, run for computer time not collisions

- **ENIAC: first calculations run April/May 1948**
  - Code finalized in **December 1947**;
  - Continuous energy neutrons, fission spectra and XS tabulated at interval mid-points, histogram energy-dependence of XS, pseudo-RN.

Thomas Haight, et al., "Los Alamos Bets on ENIAC: Nuclear Monte Carlo Simulations, 1947-1948, IEEE Ann. Of History of Comp July-Sept 2014

R.D Richtmyer and J. von Neumann "Statistical Methods in Neutron Diffusion", Los Alamos (LAMS-557) April 9, 1947.

April 2, 1947

Professor John vonNeumann,
The Institute for Advanced Study,
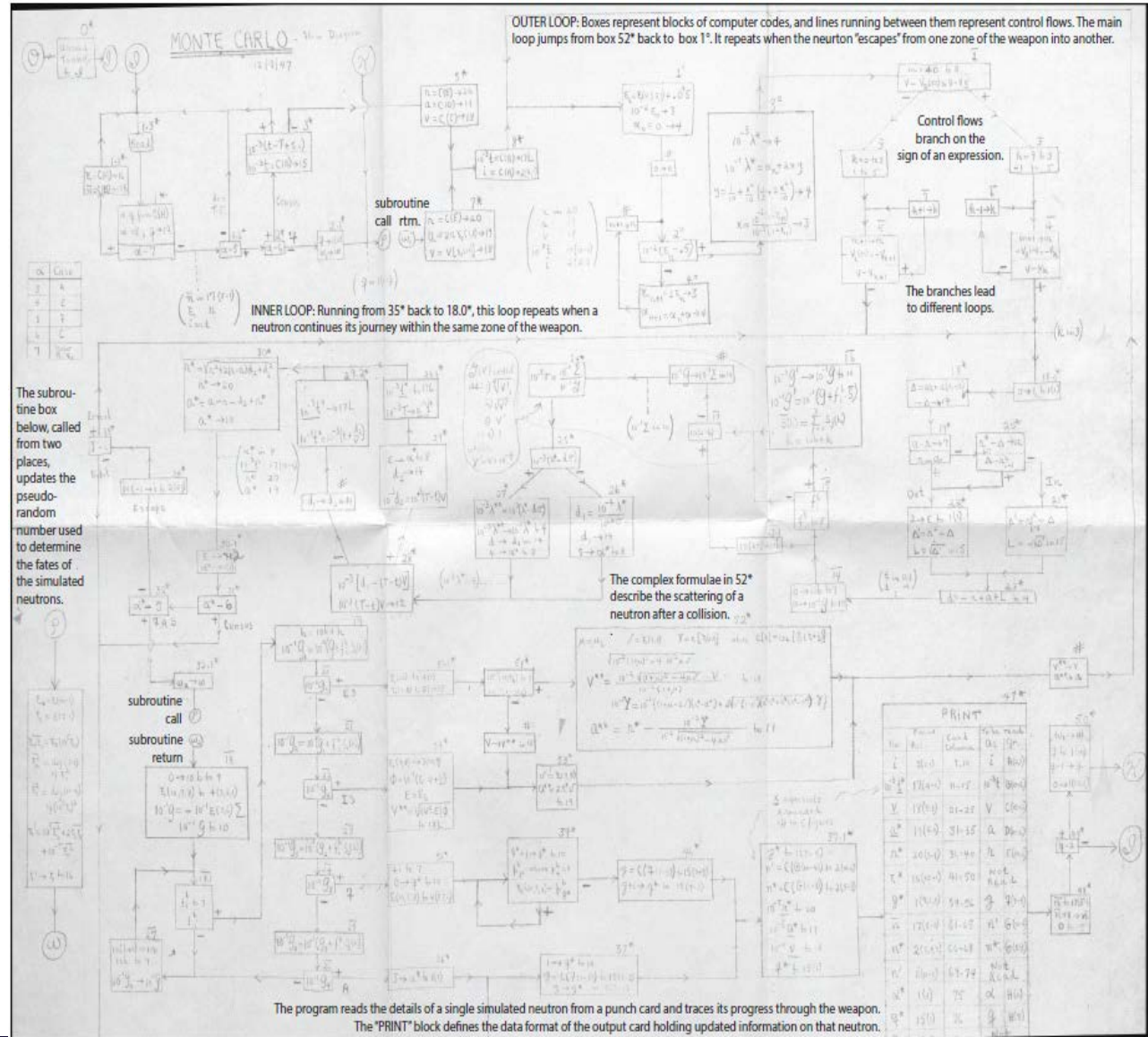School of Mathematics
Princeton, New Jersey

Dear Johnny:

As Stan told you, your letter has aroused a great deal of interest here. We have had a number of discussions of your method and Bengt Carlson has even set to work to test it out by hand calculation in a simple case.

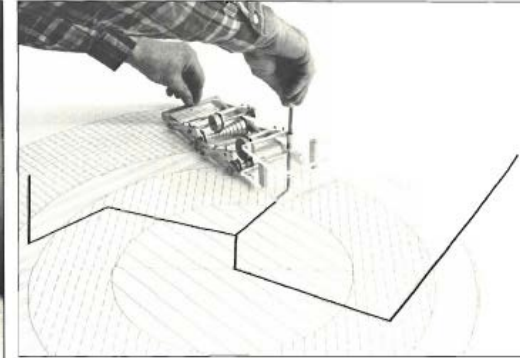| Boxes	Function
1* – 8*	Read a card and store neutron characteristics
1° – 4°	Calculate random parameter λ*
$\overline{1} - \overline{7}$	Find neutron's velocity interval
18* – 23*	Calculate distance to zone boundary
$\overline{14} - \overline{17.1}$, 24*	Calculate cross-section of material in zone
25* – 27*	Determine if terminal event is collision or escape
28* – 30*	Determine if a census comes first
31* – 35*	Discriminate between terminal events
Subroutine p/ω	Refresh random number
$\overline{18} - \overline{27}$	Determine collision type
51* – 52*	Elastic scattering
53* – 54*	Inelastic scattering
36* – 39*, 46*	Absorption/fission
37.1*, 47* – 50*	Print card and restart main loop

Thomas Haight, Mark Priestley, and Crispin Rope, "ENIAC in Action: Making and Remaking the Modern Computer," MIT Press 2016

OUTER LOOP: Boxes represent blocks of computer codes, and lines running between them represent control flows. The main loop jumps from box 52* back to box 1*. It repeats when the neutron "escapes" from one zone of the weapon into another.

Control flows branch on the sign of an expression.

The branches lead to different loops.

INNER LOOP: Running from 35* back to 18.0*, this loop repeats when a neutron continues its journey within the same zone of the weapon.

The subroutine box below, called from two places, updates the pseudo-random number used to determine the fates of the simulated neutrons.

subroutine call

subroutine return

The complex formulae in 52* describe the scattering of a neutron after a collision.

PRINT

The program reads the details of a single simulated neutron from a punch card and traces its progress through the weapon. The "PRINT" block defines the data format of the output card holding updated information on that neutron.
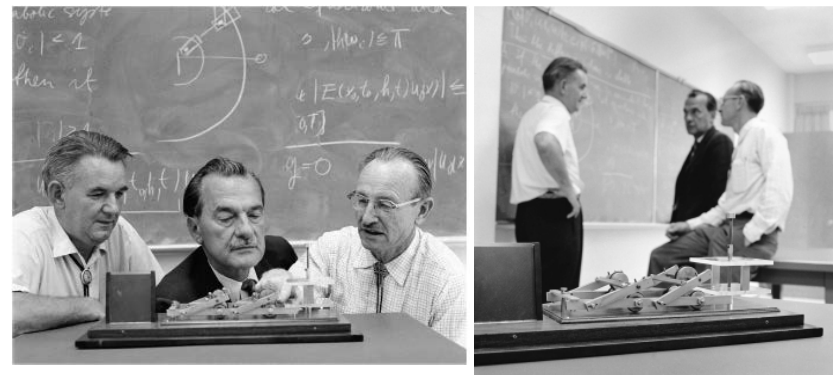
Enrico Fermi: Independently developed Monte Carlo!

- **Emilio Segre, Fermi's student and collaborator:**
  - "*Fermi had invented, but of course not named, the present Monte Carlo method when he was studying the moderation of neutrons in Rome. He did not publish anything on the subject, but he used the method to solve many problems with whatever calculating facilities he had, chiefly a small mechanical adding machine*"

- **Astonished Roman colleagues when he would predict experimental results remarkably accurately. He revealed that he used statistical sampling techniques whenever insomnia struck.**

- **15 years prior to Ulam**

- **While in Los Alamos and awaiting ENIAC's move, he created an analog device to study neutron transport.**
  - Called FERMIAC
  - Generated the site of next collision based upon characteristics of material; Another choice was made at boundary crossing; "slow" and "fast" neutron energies

Enrico Fermi

FERMIAC

Los Alamos Scientists: Bengt Carlson, Nicholas Metropolis, LDP King with Fermiac (1966)

MANIAC – Nicholas Metropolis

- **Post-war ENIAC started a revolution that continues today**
- **MANIAC – Mathematical and Numerical Integrator and Computer**
  - Was a product of Nicholas Metropolis at LANL; borrowed concepts from von Neumann's IAS, operational in 1952;
  - MADCAP – high-level language and compiler
  - Rapid growth of computing: AVIDAC (Argonne) ORACLE (Oak Ridge), ILLIAC (U of I)
  - Special effort that helped bind Von Neumann, Fermi, Beta, Teller, Ulam, Feynman, etc in post-war efforts. MANIAC was a fascination.
  - First time "Monte Carlo" appears in publication:
    - Nicholas Metropolis and S. Ulam , "The Monte Carlo Method," *Journal of the American Statistical Association* Vol. 44, No. 247 (Sep., 1949)
  - MC on MANIAC used for multiple problems other than radiation transport:

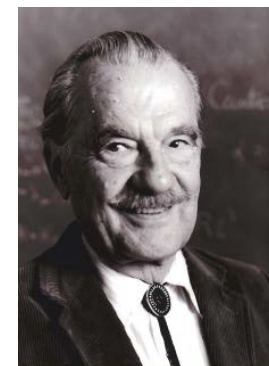JOURNAL OF THE AMERICAN STATISTICAL ASSOCIATION

Number 247   SEPTEMBER 1949   Volume 44

THE MONTE CARLO METHOD

Nicholas Metropolis and S. Ulam
Los Alamos Laboratory

We shall present here the motivation and a general description of a method dealing with a class of problems in mathematical physics. The method is, essentially, a statistical approach to the study of differential equations, or more generally, of integro-differential equations that occur in various branches of the natural sciences.

Pion-proton phase-shift analysis (Fermi, Metropolis; 1952)

Phase-shift analysis (Bethe, deHoffman, Metropolis; 1954)

Nonlinear coupled oscillators (Fermi, Pasta, Ulam; 1953)

Genetic code (Gamow, Metropolis; 1954)

Equation of state: Importance sampling (Metropolis, Teller; 1953)

Two-dimensional hydrodynamics (Metropolis, von Neumann; 1954)

Universalities of iterative functions (Metropolis, Stein, Stein; 1973)

Nuclear cascades using Monte Carlo (Metropolis, Turkevich; 1954)

Anti-clerical chess (Wells; 1956)

The lucky numbers (Metropolis, Ulam; 1956)

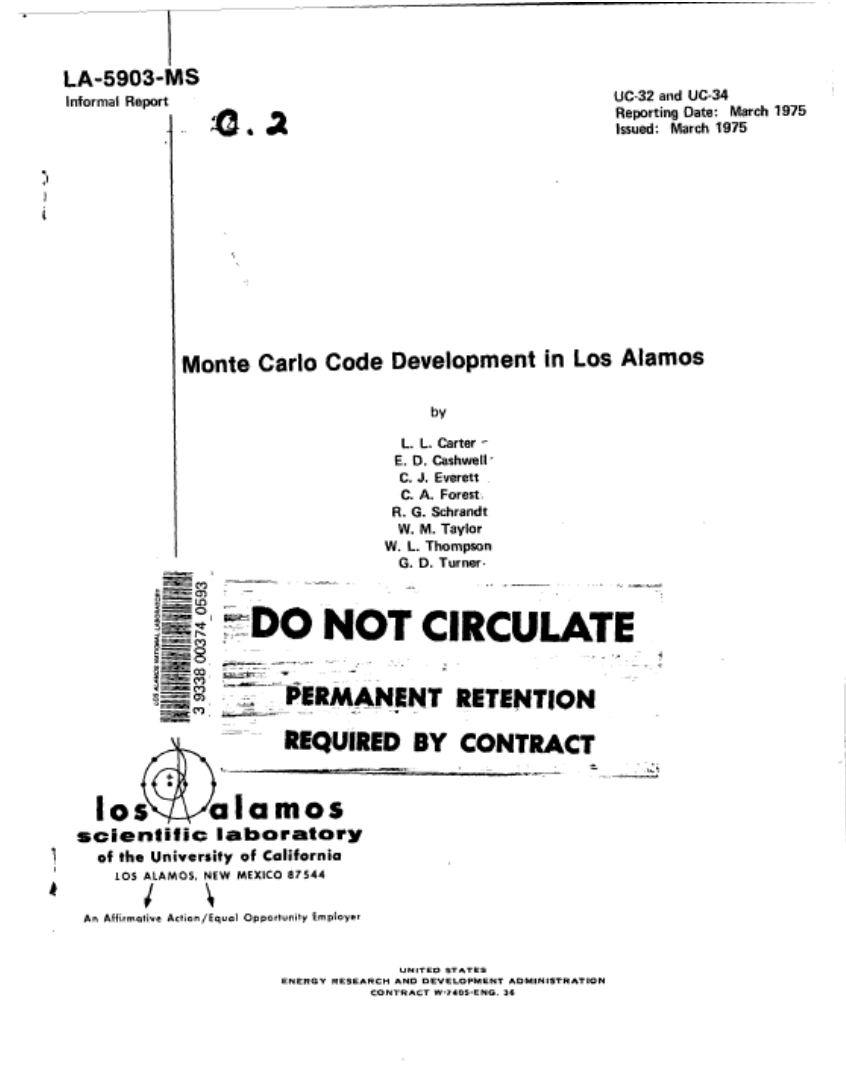# 1950's: Monte Carlo is becoming mainstream!

- **Herman Kahn, "Applications of Monte Carlo," AECU-3259 (April 19, 1954).**
  - General and not specific to radiation particle transport
  - Direct sampling for common distributions
  - Rejection sampling – if direct sampling does not work
    - Use a simple, easy to sample distribution, to get an estimate and correct later.
  - Russian Roulette
  - Stratified sampling, importance sampling, splitting
- **E.D. Cashwell and C.J. Everett, "A Practical Manual on the Monte Carlo Method for Random Walk Problems," LA-2120 (December 18, 1957)**
  - Well-described report specific to particle transport
  - Detailed diagrams and flowcharts
    - Neutron collisions – (in)elastic scattering, fission, etc
    - Photon collisions – Compton scattering, photoelectric, pair prouction
    - Particle direction after collision – direction cosines
  - Did not deal with thermal neutron collisions nor pseudo-random number generation

Thomas Sutton and David Griesheimer, "The Monte Carlo Method: The Past 70 Years, Current State and Future Prospects," ANS Math and Computational Division Topical (Jeju, South Korea, April 2017)

1960's: Initial work on Criticality and Reactor Calculations

- **JJ.B. Parker, and E.R. Woodcock, "Monte Carlo Criticality Calculations," Prog. In Nucl. Ener, 4 (1961).**
  - Introduced concept of neutron generations or batches of particles as histories
- **E.R. Woodcock, T. Murphy, P.J. Hemmings, and T.C. Longworth, "Techniques Used in the GEM Code for Monte Carlo Neutronics Calculations in Reactors and Other Systems of Complex Geometry," ANL-1050 (1965).**
  - Describes "Woodcock tracking" (aka delta-tracking)
    - Regular tracking on geometries with many surfaces is expensive; esp for 2nd order surfaces
    - Avoids multiple distance-to-boundary calculations by using fictitious XS and adjusting
- **J. Lieberoth, "A Monte Carlo Technique to Solve the Static Eigenvalue Problem of the Boltzmann Transport Equation," Nukleonik 11, 213-219 (1968)**
- **M.R. Mendelson, "Monte Carlo Criticality Calculations for Thermal Reactors," Nucl. Sci. Eng 32, 319-331 (1968).**
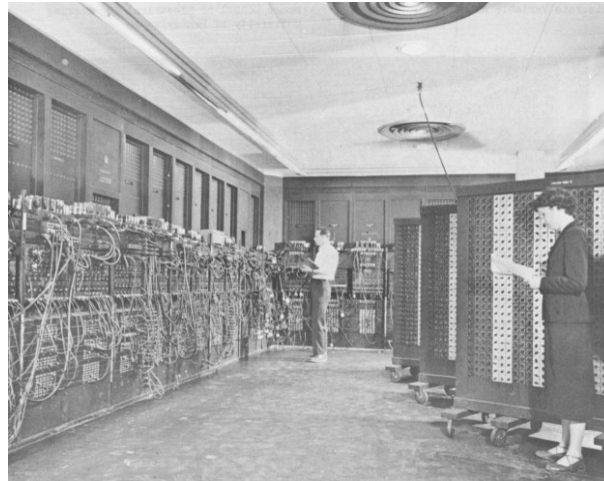
1970: MCNP first emerges

- **1974 – NEA Committee on Reactor Physics (NEACRP)**
  - E.D. Cashwell presented a paper on Monte Carlo development at Los Alamos
    - MCN – neutrons
    - MCNA – neutron adjoint
    - MCG – gamma rays
    - MCP – general photons
    - MCNG – coupled neutron-gamma ray
    - MCMG – multi-group coupled neutron-gamma ray
    - MCGE – coupled electron-photon
    - MCGB – gamma rays with Bremsstrahlung
- **1977: MCNG was merged with MCP to form MCNP**
  - 2017 is the 40th Anniversary of MCNP
  - 2017 is also the 70th Anniversary of the Monte Carlo method

LA-5903-MS
Informal Report

UC-32 and UC-34
Reporting Date: March 1975
Issued: March 1975

Monte Carlo Code Development in Los Alamos

by

L. L. Carter
E. D. Cashwell
C. J. Everett
C. A. Forest
R. G. Schrandt
W. M. Taylor
W. L. Thompson
G. D. Turner

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UNITED STATES
ENERGY RESEARCH AND DEVELOPMENT ADMINISTRATION
CONTRACT W-7405-ENG. 36

Monte Carlo & MCNP History

ENIAC – 1945

30 tons

20 ft x 40 ft room

18,000 vacuum tubes

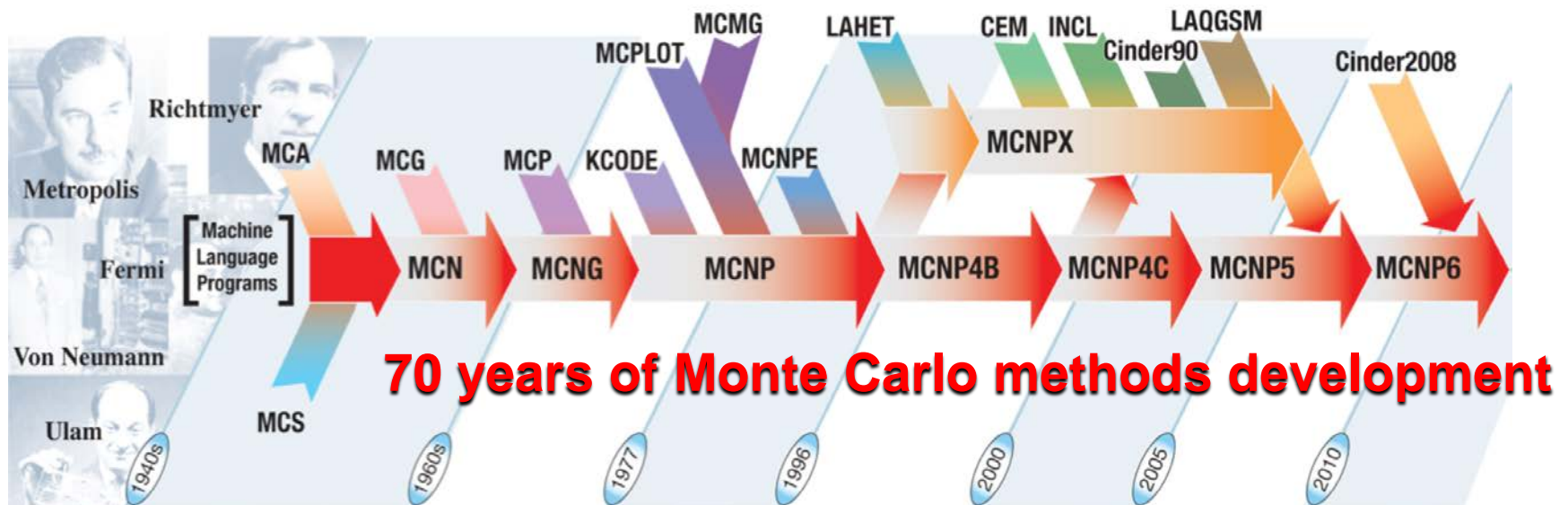0.1 MHz

20 word memory

patchcords

Manhattan Project – 1945...

Discussions on using ENIAC

Ulam suggested using the "method of statistical trials"

Metropolis suggested the name "Monte Carlo"

Von Neumann developed the first computer code

70 years of Monte Carlo methods development

1980 – 1999: MCNP Becomes the Gold Standard

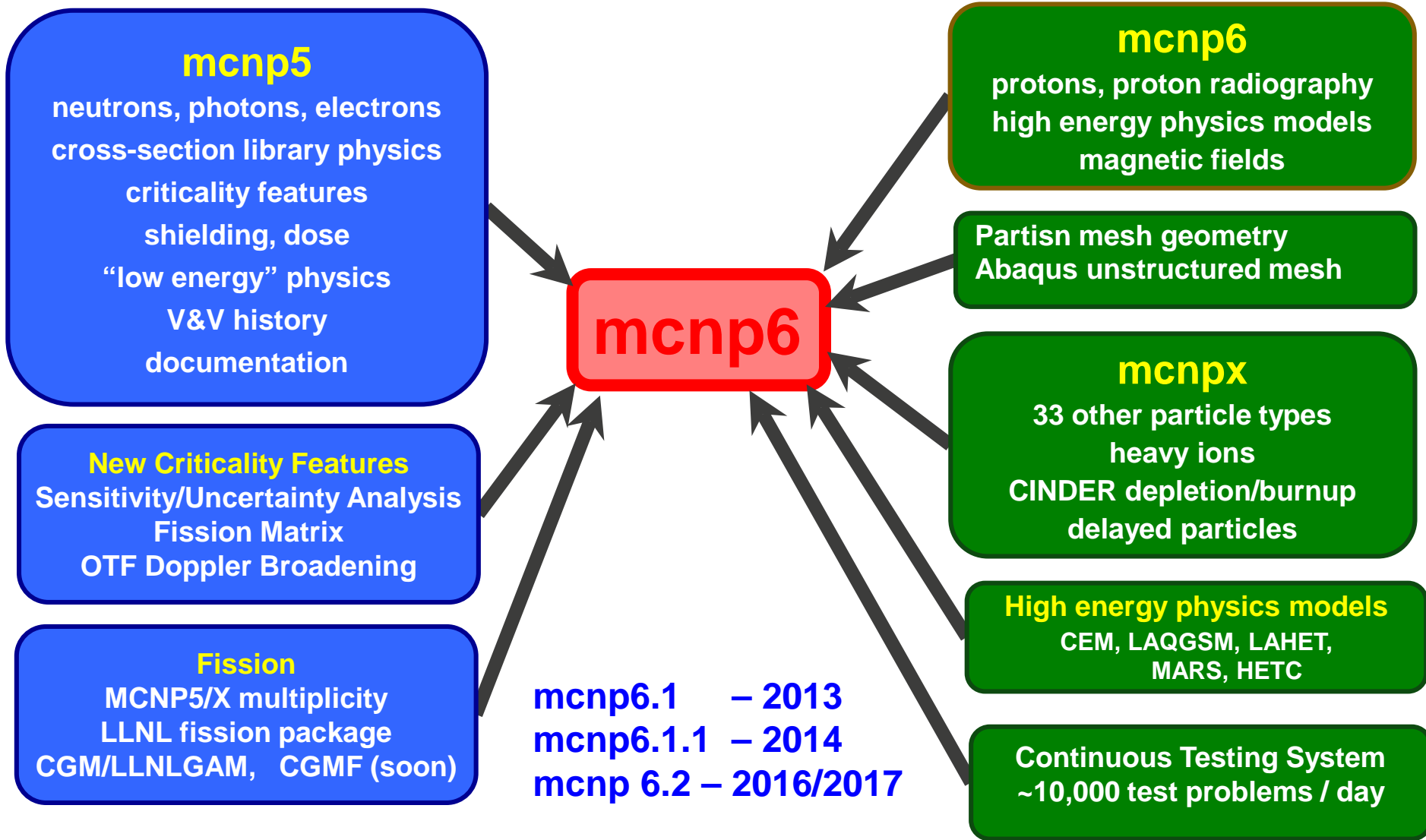| MCNP Version	Release Month/Year	Some Significant New Features (For a more detailed description, see each version's release notes).
MCNP3	1983	First release through RSICC. Written in Fortran 77
MCNP3A	1986	
MCNP3B	1988	Plotting graphics, generalized source, surface sources, repeated structures/lattice geometries
MCNP4	1990	Parallel multitasking, electron transport
MCNP4A	10/1993	Enhanced statistical analysis, new photon libraries, ENDF-6, color X-Windows graphics, dynamic memory allocation
MCNP4B	4/1997	Operator perturbations, enhanced photon physics, PVM load balancing, cross-section plotting, 64-bit executables, lattice universe mapping, enhanced lifetimes
MCNPX 2.1.5	11/1999	First public release of MCNPX, based on MCNP4B with CEM INC, HTAPE3X, mesh and radiography tallies, and an improved collisional energy loss model.

T. Goorley, et al. "Initial MCNP6 Release Overview," LA-UR-13-22934 (2013)

- **Key Value: MCNP provides a predictive capability that can replace expensive or impossible-to-perform experiments**
- **Used to design large-scale measurements providing significant time/cost savings**
- **MCNP represents a synergistic capabilities developed at LANL**
  - Evaluated nuclear data (ENDF) and data processing code NJOY
  - MCNP could not exist without this!
- **International user community's high confidence in MCNP's predictive capabilities are based upon its performance with verification and validation test suites.**

2000 – 2011: MCNP undergoes exponential growth

MCNP4C	4/2000	Unresolved resonance treatments, macrobodies, superimposed importance mesh, perturbation, electron transport, plotter and tally enhancements
MCNP4C2	1/2001	Photonuclear physics, interactive plotting, plot superimposed weight-window mesh, weight-window improvements
MCNPX 2.3.0	4/2002	LAHET 2.8 and some 3.0 extensions.
MCNPX 2.4.0	8/2002	Update to MCNP4C, build system for Windows OS, support for Fortran 90.
MCNP5 1.14	11/2002	Fortran 90, photonuclear collisions, geometry superimposed mesh tallies, time splitting, shared memory threading with OpenMP. Mac OSX support
MCNP5 1.20	10/2003	Increased number of detectors to 100 and number of tallies to 1000. Mostly a code defect fix release.
MCNP5 1.30	8/2004	Explicit 8-byte integers for nps > 2.1 billion, Lattice and fmesh tally enchantments. Support for MPI on Mac OSX.
MCNPX 2.5.0	4/2005	34 particle types, four light ions, mix and match nuclear data tables and model physics, CEM2k, INCL4/ABLA physics models, fission multiplicity, spontaneous fission sources, pulse height tallies with variance reduction, pulse height light tally, coincident capture tallies, variance reduction with model
MCNP5 1.40	11/2005	Lethargy plots, logarithmic data interpolation, neutron multiplicity distributions, stochastic geometry, source entropy, mesh tally plots, new electron energy loss straggling
MCNPX 2.6.0	4/2008	Depletion/Burnup, heavy ion transport, LAQGSM physics, CEM03 physics, delayed gamma emission, energy-time weight windows, charged ions from neutron capture, spherical mesh weight windows, spontaneous photons
MCNP5 1.51	1/2009	Photon Doppler broadening, variance reduction with pulse height tallies, annihilation gamma tracking, Doppler broadening in makxsf, large lattice enhancements
MCNP5 1.60	8/2010	Adjoint weighted tallies for point kinetics parameters, mesh tallies for isotopic reaction rates, up to 100 million cells & surfaces, up to 10 thousand tallies
MCNPX 2.7.0	4/2011	Tally Tagging, embedded sources, cyclic time bins, focused beam sources, PTRAC coincidence, LLNL fission multiplicity, Receiver-operator characterization (ROC) tally, NRF data in ACE libraries, triple & quadruple coincidence, LAQGSM 3.03 and CEM 3.03 physics.

mcnp5

neutrons, photons, electrons

cross-section library physics

criticality features

shielding, dose

"low energy" physics

V&V history

documentation

New Criticality Features
Sensitivity/Uncertainty Analysis
Fission Matrix
OTF Doppler Broadening

Fission
MCNP5/X multiplicity
LLNL fission package
CGM/LLNLGAM, CGMF (soon)

mcnp6

mcnp6
protons, proton radiography
high energy physics models
magnetic fields

Partisn mesh geometry
Abaqus unstructured mesh

mcnpx
33 other particle types
heavy ions
CINDER depletion/burnup
delayed particles

High energy physics models
CEM, LAQGSM, LAHET,
MARS, HETC

mcnp6.1   – 2013
mcnp6.1.1 – 2014
mcnp 6.2 – 2016/2017

Continuous Testing System
~10,000 test problems / day

MCNP® Capabilities

- **Physics:**
  - Continuous energy particle transport
  - Neutron, photon, electron, and many more particle types

- **Algorithms:**
  - k-eigenvalue calculations
  - Fixed source calculations

- **Recently Implemented Features:**
  - Unstructured mesh transport
  - Electric and magnetic field transport
  - High-energy physics models
  - 33 additional particle types
  - Reactor fuel depletion and burnup
  - Radiation source and detection capabilities
  - Sensitivity and uncertainty analysis for nuclear criticality safety

- **Extensive Variance Reduction**
  - Weight Windows
  - DXTRAN

Whole-core Thermal & Total Flux

Experimental Benchmarks with Critical Assemblies

HEU-MET-THERM-003

Zeus-2, HEU-MET-INTER-006, case 2

Jezebel – K$_{eff}$ Sensitivity to PFNS
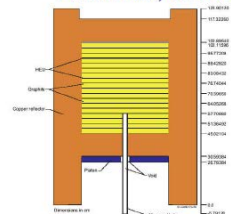
Forward Perturbation Method
MCNP6 Adjoint-Weighting

PU-MET-FAST-001
^{239}Pu(n,f) PFNS

Jezebel K$_{eff}$ / PFNS Relative Sensitivity (per unit lethargy)

Outgoing Neutron Energy (MeV)

beam

ITER Neutron Flux Calculations

City model used to study nuclear weapon effects

Monte Carlo Codes from across the globe

- **Monte Carlo Codes Session at SNA+MC 2013**
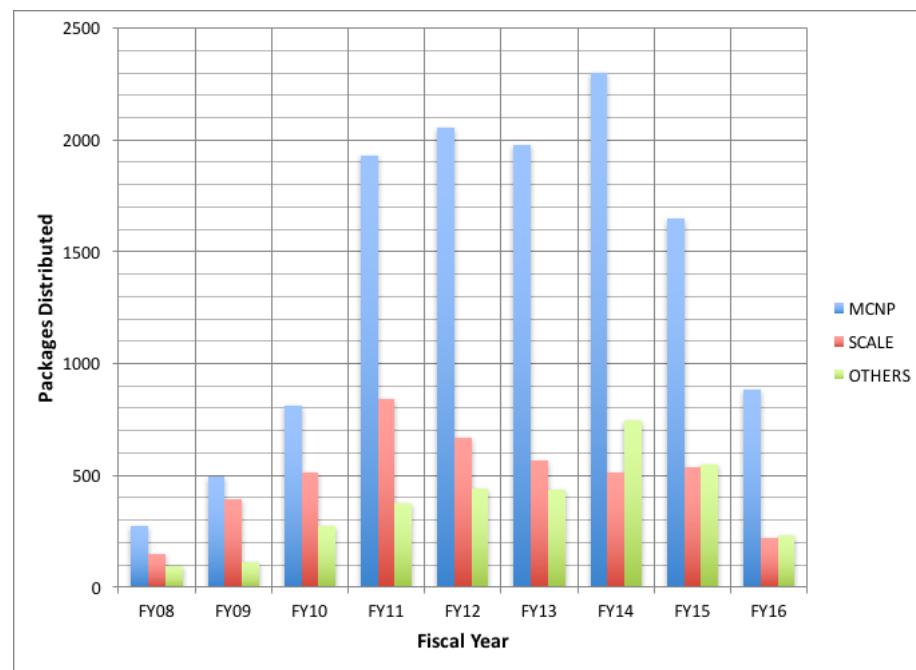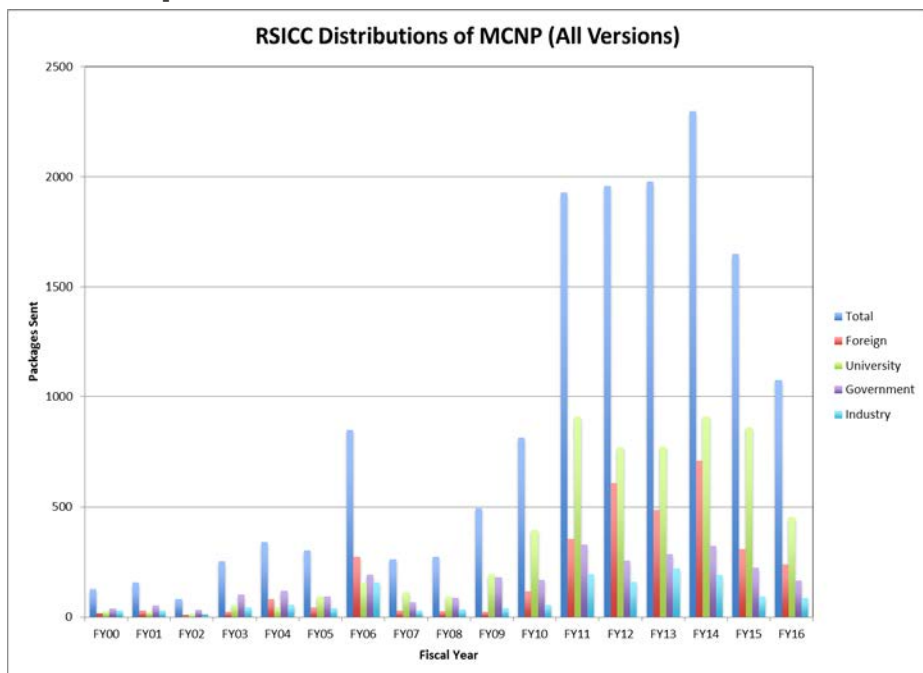 - Annals of Nuclear Energy, 82 (2015)

code	institution	
ARCHER	RPI	USA
COG11	LLNL	USA
DIANE	CEA	France
FLUKA	INFN & CERN	Italy & CERN
GEANT4	GEANT4	International
KENO and MONACO	ORNL	USA
MC21	Naval Nucl. Lab.	USA
MCATK	LANL	USA
MCCARD	Seoul Natl. Univ.	ROK
MCNP6	LANL	USA
MCU	Kurchatov Inst.	Russia

code	institution	
MONK & MCBEND	AMEC Foster Wheeler	UK
MORET5	IRSN	France
MVP2	JAEA	Japan
OPENMC	MIT	USA
PENELOPE	Barcelona Univ.	Spain
PHITS	JAEA	Japan
PRIZMA	VNIITF	Russia
RMC	Tsinghua Univ.	China
SERPENT	VTT	Finland
SUPERMC	CAS INEST FDS	China
TRIPOLI-4	CEA	France

MCNP Distribution: RSICC

- **MCNP is export controlled and is distributed:**
  - USA: RSICC – Oak Ridge National Laboratory, LANL
  - Europe: NEA Databank; Japan, Korea: KAIST – coordinated through RSICC
- **Approximately 20,000 copies of MCNP licenses have been distributed.**
  - 8000 copies of MCNP 6 since 2011 (Data provided by T. Valentine, RSICC)

  Today: All requests eventually are through RSICC or LANL with appropriate DOE / export control reviews

MCNP in the near future

- **Motivation: LANL, DOE/NNSA, DHS-DNDO, and DTRA sponsors need a predictive capability**

- **Biggest needs are:**
  - Validated models of geometry and materials; complex radiation sources; direct comparison with radiation detection instruments

- **MCNP 2020 vision:**
  - Library-based Monte Carlo framework
  - Software quality improvements

- **Applications:**
  - Next-generation high performance computers
  - Multi-physics: MCNP often needed within other scientific software
  - Tools to assist users
    - Geometry: Collaborations with industry
      - Allow users to take CAD/CAE, modify and develop mesh-based models; Variance reduction with Sn
    - Radiation Source: ISC – generalized intrinsic source (aged) from any decay library
    - Transport physics: Correlated source/collision physics
    - Tallies: MCNP tools – a package to facilitate user access to MCNP output
      - Users can produce tools to make plots, analyze data, etc without headache of having to parse data
      - One application: radiation detector response

Thank you!

Contact Info:
Avneet Sood
sooda@lanl.gov