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OECD-NEA-WPNCS
Expert Group meeting

Advanced Monte Carlo
Techniques

Paris, June 2017

Investigation of Clustering in MCNP6 Monte Carlo Criticality Calculations

LA-UR-17-

Forrest Brown

Monte Carlo Methods, Codes, & Applications (XCP-3)
X Computational Physics Division



Introduction

- **Monte Carlo**
 - Simulate particle behavior
 - Tally event occurrences to estimate physical results
 - Must have enough particles to cover phase space of the problem
- **The undersampling problem**
 - Not enough particles to cover phase space
 - All MC results are questionable, possibly wrong
 - How can you diagnose the absence of coverage ?
 - The cure: Run more particles in the simulation
 - Questions: How many? How do you know it's enough?
- **Clustering**
 - For criticality problems
 - Iterations using next-generation fission source
 - Convergence assessment depends on fission source coverage
 - In some problems, repeated iterations lead to clustering

Sutton's Model Problem & Shannon Entropy

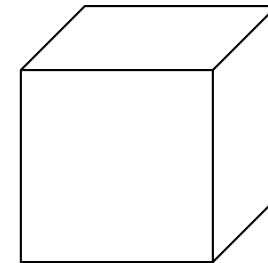
Sutton's Model Problem

Recent references

- T.M. Sutton & A. Mittal, "Neutron Clustering in Monte Carlo Iterated-Source Calculations", ANS MCD 2017, Jeju, S. Korea, April 16-20, 2017
- A. Zoia, E. Dumonteil, "Neutron clustering: spatial fluctuations in multiplying systems at the critical point", ANS MCD 2017, Jeju, S. Korea, April 16-20, 2017

Model problem for clustering investigations

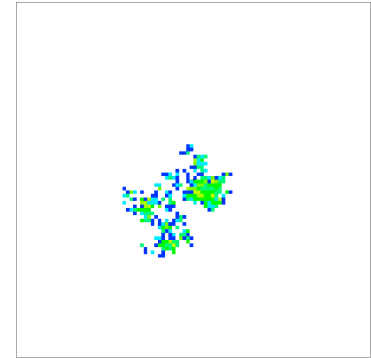
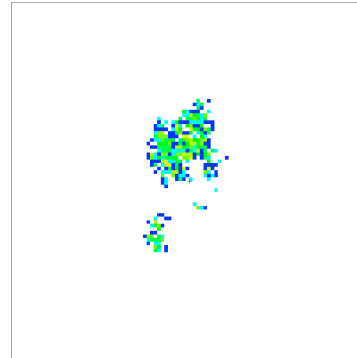
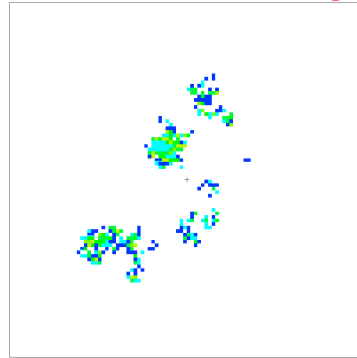
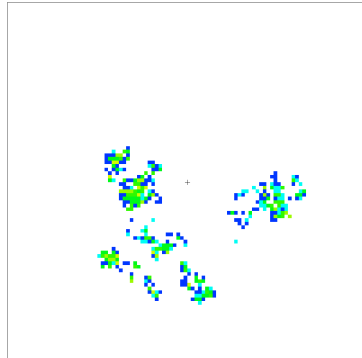
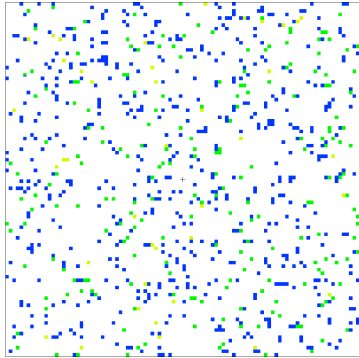
- Homogeneous box
- 400 x 400 x 400 cm³
- reflecting boundary conditions
- One-speed: $\Sigma_T = 1.0$, $\Sigma_S = 0.6$, $\Sigma_C = 0.2$, $\Sigma_F = 0.2$, $\nu = 2.4$, $f(\mu) = \frac{1}{2}$



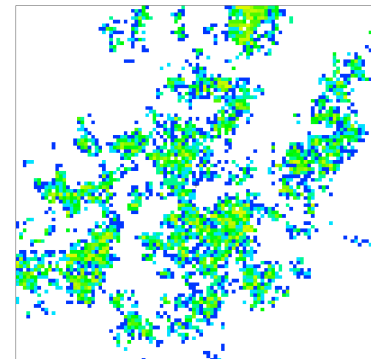
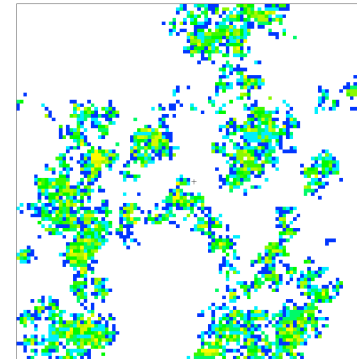
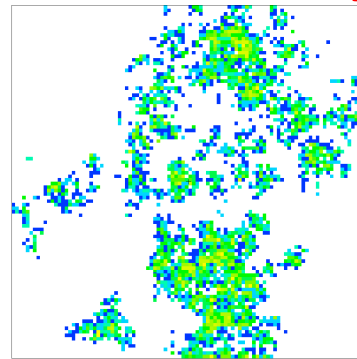
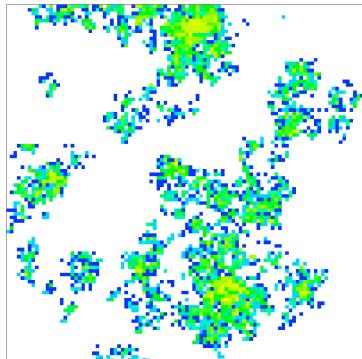
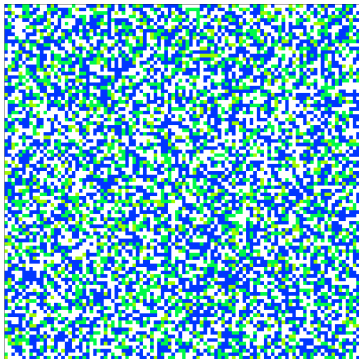
- **Exact solution:** uniform distribution of fission sites throughout volume of box
 - Start with initial source guess = exact solution, uniform in volume
 - Shannon entropy for exact uniform source distribution: $H_{\text{exact}} = \log_2(N_s)$, where N_s is the number of grid-cells in Shannon entropy mesh
 - **For a 10 x 10 x 10 Shannon entropy mesh, $H_{\text{exact}} = \log_2(100) = 9.966$**
 - Can compare actual H_{src} for calculations that vary some of the problem parameters to H_{exact} , as an indicator of clustering in this model problem

Clustering vs Neutrons/cycle

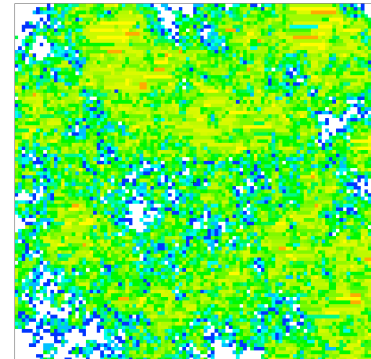
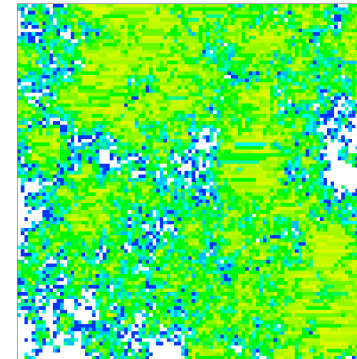
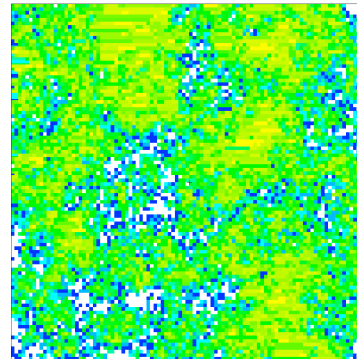
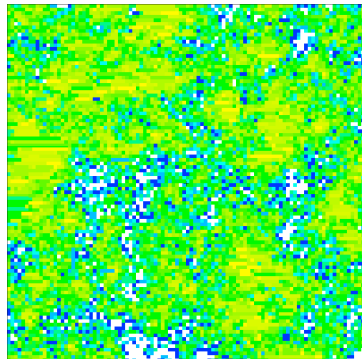
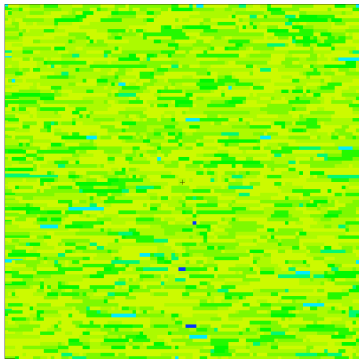
1000 neutrons/cycle



10,000 neutrons/cycle



100,000 neutrons/cycle



Cycle 1

Cycle 1000

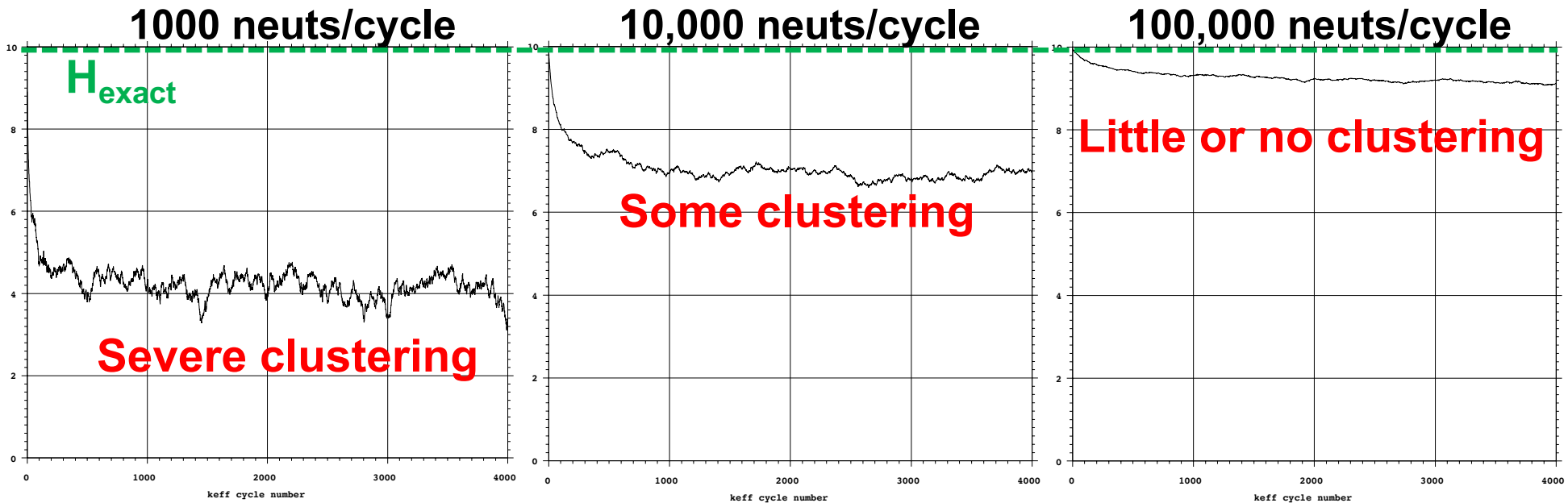
Cycle 2000

Cycle 3000

Cycle 4000

Clustering and Shannon Entropy

- Shannon entropy vs cycle



– For this model problem (running 5000 cycles)

- Visual inspection of plots of fission source points
- MCNP determination of H_{ave} for the last half of the problem

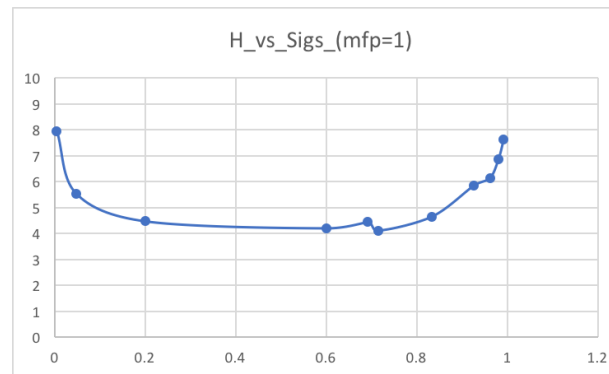
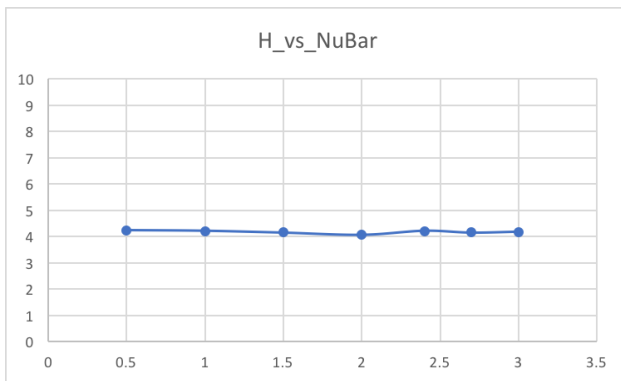
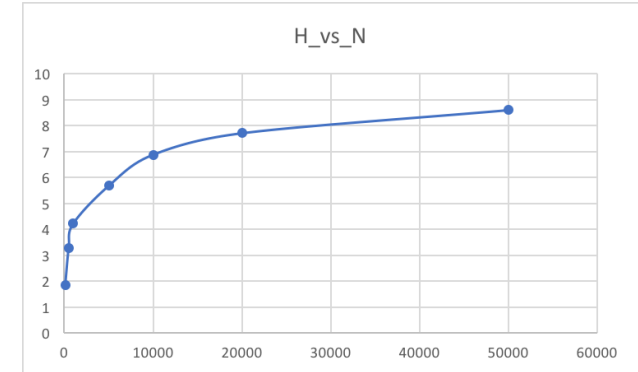
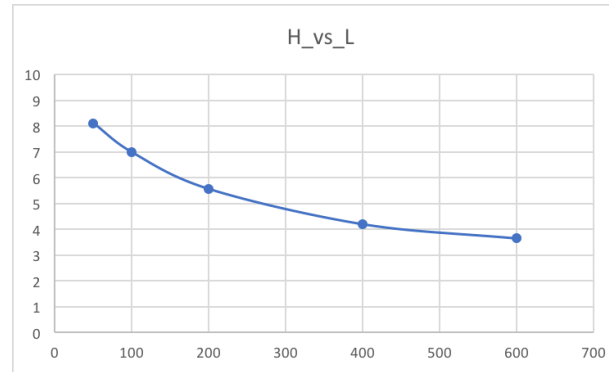
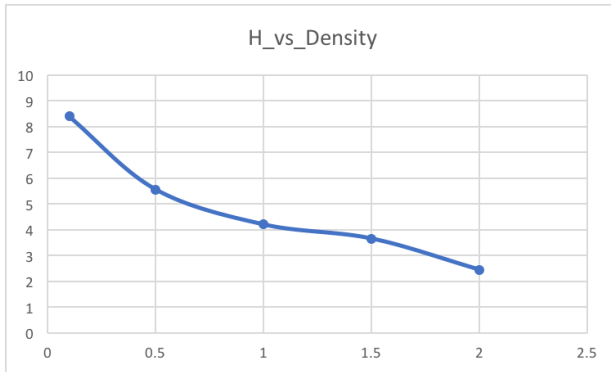
$$H_{\text{ave}} < 0.7 H_{\text{exact}}$$

corresponds to **severe** clustering

$$H_{\text{ave}} > 0.7 H_{\text{exact}}$$

corresponds to **some or no** clustering

H vs Varying Parameters



Note that cases with same ρL or same mfp/L have identical clustering

That is, $2*\rho$ and $.5*L$ (or $.5*mfp$ and $.5*L$) does not change clustering

(Remember that this is an infinite medium, no leakage)

**Higher density
Larger size
Small neut/cycle
Smaller mfp** → **lower H, more clustering**

A Simple Physical Approach

- **For the original problem**

- $\lambda = 1.00$ cm
- $\ell_F = 2.23$ cm, RMS distance from birth to fission site (from mcnp6)
- $L = 400$ cm

– So,

If a single neutron “covers” a volume $(4\pi/3 \cdot \ell_F^3)$,
and for this problem total volume = L^3

max coverage for

1,000 neut	~	0.073 %	of volume	-	severe clustering
10,000 neut	~	0.73 %	of volume	-	some clustering
100,000 neut	~	7.3 %	of volume	-	no clustering

define $f_H^{\max} = \text{max fraction of H volume covered, } N \cdot (4\pi/3 \cdot \ell_F^3) / V_H$

(assumes no overlap of spheres, so can be >100%)

Clustering and Shannon Entropy (more)

• Shannon entropy

– Used to diagnose convergence of iterated fission source

- Superimpose coarse mesh, $N_s = m \times m \times m$ bins
- For each iteration, tally N fission neutrons in bins
- Normalize to get $\{ p_k, k=1, \dots, N_s \}$, coarse global PDF
- Then,

$$H = - \text{Sum } p_k \log_2(p_k), \quad \text{note: } 0 \log_2(0) = 0$$

Uniform particle distribution \rightarrow max H : $H_{\max} = \log_2(N_s)$

All neutrons at same point \rightarrow min H : $H_{\min} = 0$

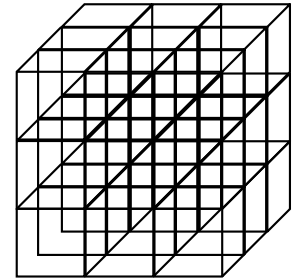
- Plot H vs cycle, converged when H is asymptotically constant

– Fundamental assumption:

$N \gg N_s$, enough neutrons to get reliable p_k tallies

• Clustering reduces the computed Shannon entropy

- If N is small, coverage is not sufficient for reliable p_k tallies
- If $N \sim N_s$ or $N < N_s$, $H_{\max} = \log_2(N)$, wrong!



Clustering and Shannon Entropy (more)

- **Shannon entropy**

$$H = - \text{Sum } p_k \log_2(p_k), \quad \text{note: } 0 \log_2(0) = 0$$

- For $N_S = m \times m \times m$ bins, and N neutrons

- Uniform particle distribution: $H_{\max} = \log_2(N_S)$
- All neutrons at same point: $H_{\min} = 0$

- **Simple example**

- $10 \times 10 \times 10$ mesh, $N_S = 1000$

- For $N = 1,000$ neutrons

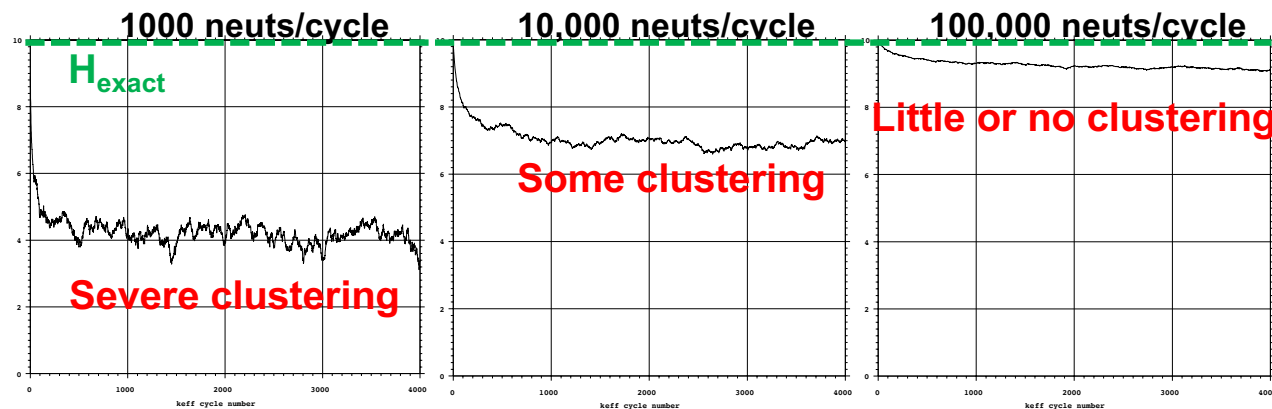
1 neut/bin, uniform	$H = 9.97$	
2 neuts/bin, 0 in others	$H = 8.97$	500 clusters of 2
4 neuts/bin, 0 in others	$H = 7.97$	250 clusters of 4
8 neuts/bin, 0 in others	$H = 6.97$	125 clusters of 8
125 neuts/bin, 0 in others	$H = 3.00$	8 clusters of 125
250 neuts/bin, 0 in others	$H = 2.00$	4 clusters of 250
500 neuts/bin, 0 in others	$H = 1.00$	2 clusters of 500
1000 neuts/bin, 0 in others	$H = 0.00$	1 cluster of 1000

- Clustering reduces the computed Shannon entropy

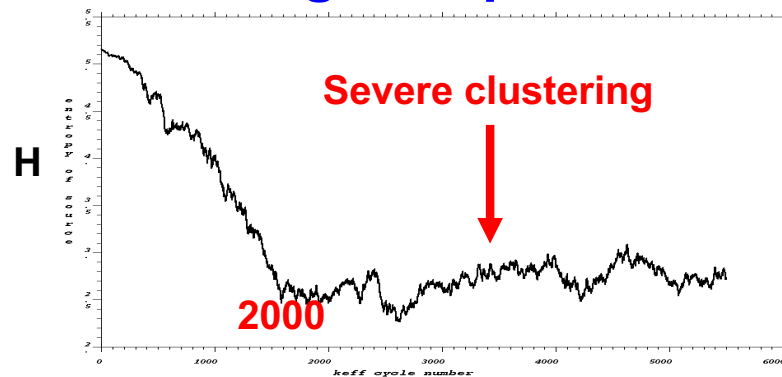
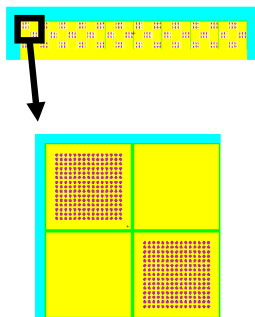
Clustering and Shannon Entropy (more)

- **Shannon entropy & clustering**

- Clustering leads to erroneously small asymptotic H , but how do you diagnose that if you don't know H_{exact} ?
- Clustering leads to jagged, gross variations in asymptotic H , which can be observed

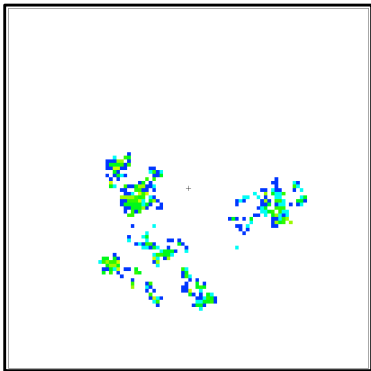


- **Remember the EG-Source-Convergence problem?**



Cluster Analysis, Using DBSCAN

- There are many algorithms for identifying clusters
 - Used in image processing, etc.
 - A simple & useful algorithm is DBSCAN (density-based scan)



```

====> file srctp
      nattrib =      10
      npts =      968
x range: 0.000000000000000E+000  400.0000000000000
y range: 0.000000000000000E+000  400.0000000000000
z range: 0.000000000000000E+000  400.0000000000000

eps =      25.000000000000000
minpts =      4
npts =      968

nclust =      4
counts for each cluster:
  1      225
  2      267
  3      94
  4      378
outliers:      4

```

```

          2      2 2 2 2
        2 2 2
      2
    3
  3 3 3 3 3
 3 3 3 3
 3 3
    2 2 2 2 2
    2 2 2 2 2 2
  0 2 2 2 2 2 2
    2 2
      2
    4 4 4
    4 4 4 4 4
    4 4 4 4 4 4
    4 4 4
  4 4 4 4 4 4
  4 4 4 4 4 4
  4 4 4 4 4
  4 4 4 4 4
  4 4 4 4
  4 4 4 4

```

```

0 0
    1 1      1      1
  1 1 1 1 1 1 1 1
    1 1 1 1 1 1 1
  1      1 1 1 1 1 1
  1 1 1 1 1 1 1 1
    1 1 1 1 1 1 1
      1      1 1
        1

```

DBSCAN applied
to original problem,
with 1000 n/cycle

View from top

4 clusters (in 3D)

Need to choose 2
parameters, eps &
minpts

It is not clear how
useful the cluster
analysis is

A Real Problem

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ICSBEP

pu-sol-therm-012-13

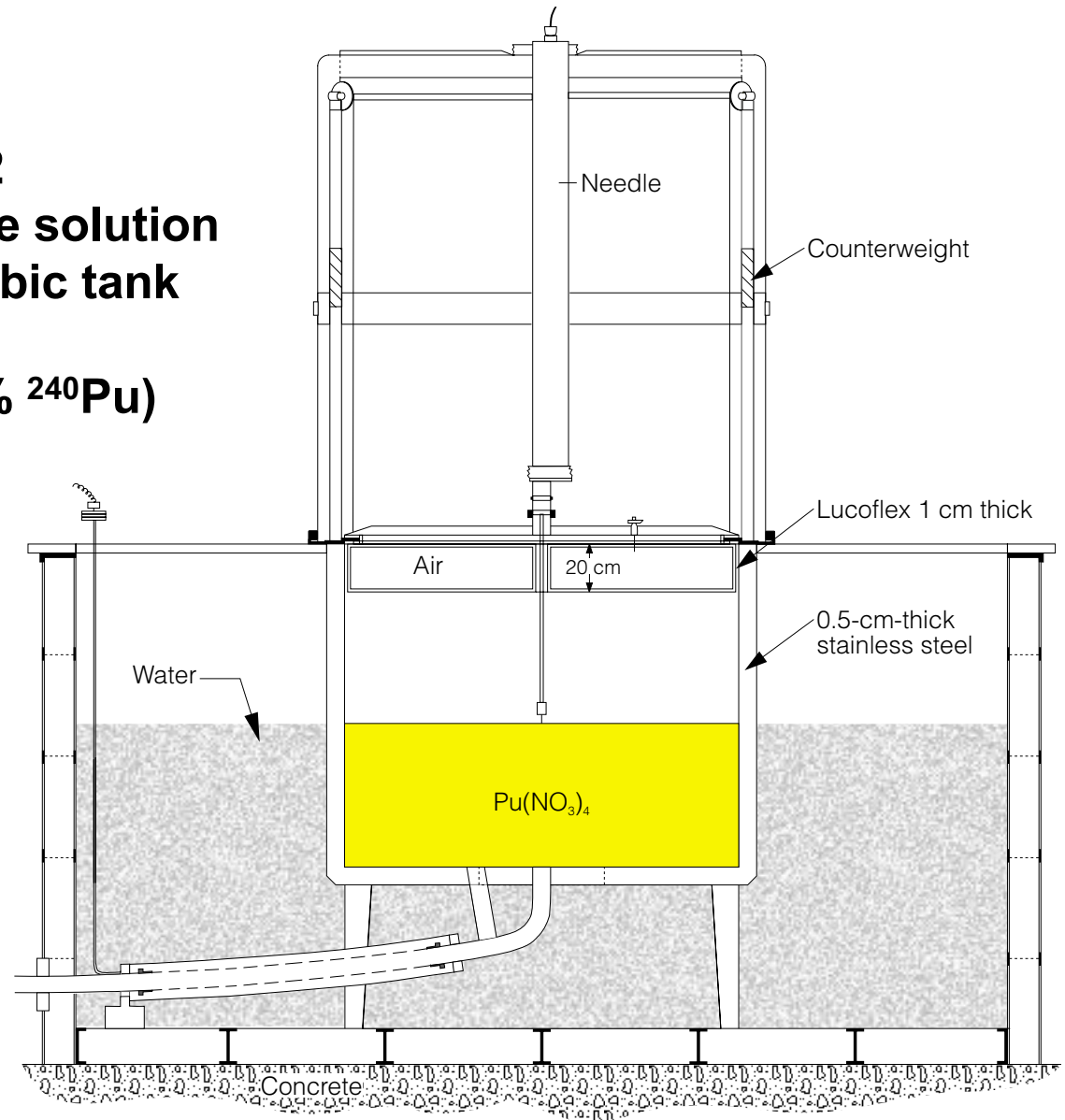
Pu-sol-therm-012 Case 13

PU-SOL-THERM-012 Criticality of plutonium nitrate solution In a large water-reflected cubic tank

(130 x 130 x 67.46 cm) (19% ^{240}Pu)

```

C Pu(NO3)4 Solution
C 13.2 gPu/cc total
C atoms= 1.00306E-01
c
M1  94239  2.47132E-05
    94240  6.26195E-06
    94241  1.85624E-06
    94242  3.74965E-07
    95241  2.01156E-07
    7014   1.37165E-03
    8016   3.53011E-02
    1001   6.35948E-02
    26000  3.55846E-06
    24000  1.14431E-06
    28000  8.11038E-07
  
```

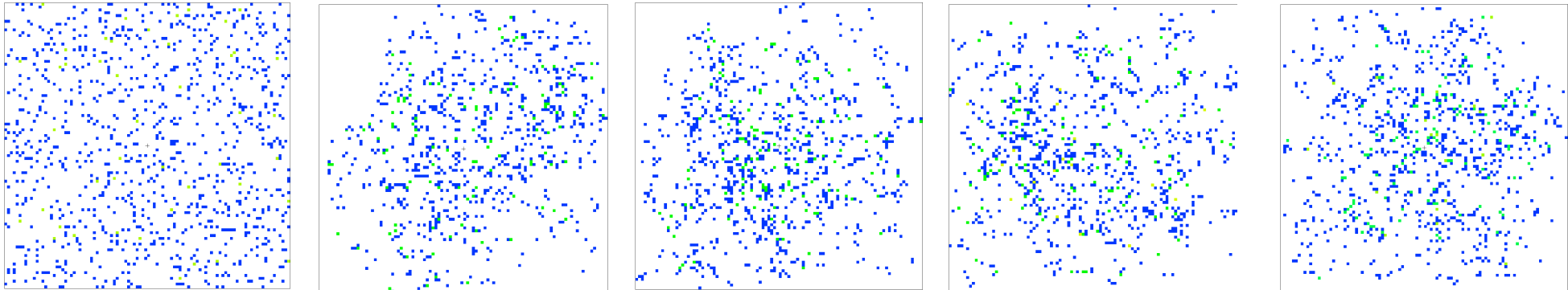


5 sides water reflected experimental configuration

Pu-sol-therm-012 Case 13

- Examine source points in fissile solution
- No clustering is evident, even with only 1,000 neutrons/cycle

1000 neutrons/cycle



Cycle 1

Cycle 1000

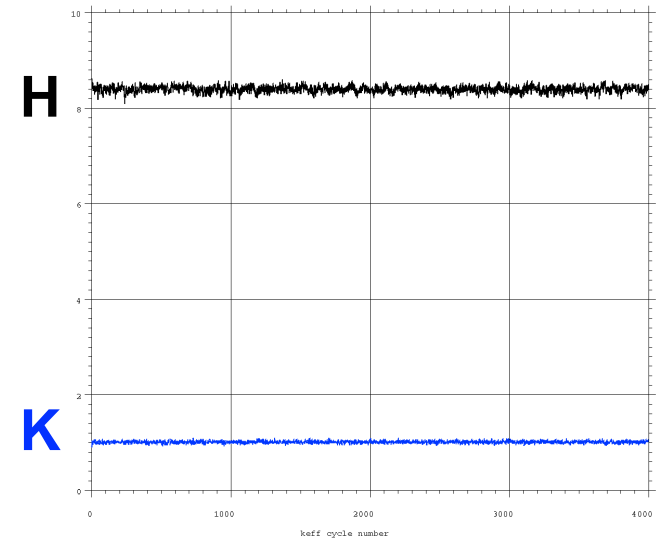
Cycle 2000

Cycle 3000

Cycle 4000

RMS distance between fissions, $\ell_F = 13.1 \text{ cm}$
 Max coverage of H_{src} volume, $f_H^{\text{max}} = 814 \%$
 Fraction of H_{src} volume with fission, $f_H = 42 \%$
 $\ell_F / (\text{mean chord length}), \ell_F / \ell_{\text{geom}} = 20 \%$

cycles to coalesce to 1 chain = 1228

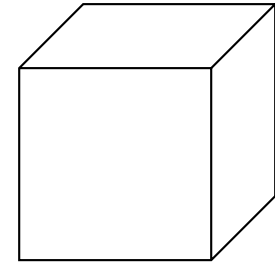


Sutton's Model Problem Using Solution from pu-sol-therm-012-13

Model Problem, with pu-sol-therm-012-13 Solution

- **Model problem for clustering investigations**

- Homogeneous box
- 400 x 400 x 400 cm³
- reflecting boundary conditions
- Material: **fissile solution from pu-sol-therm-012-13**

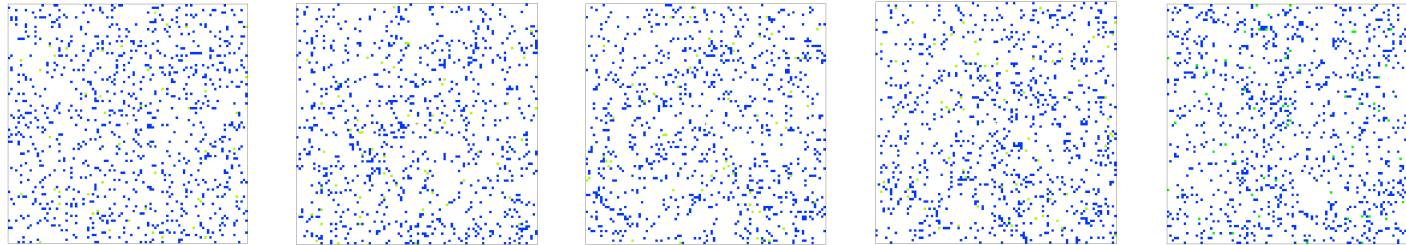


- **Note that the volume is ~56x larger than pu-sol-therm-012-13**
- **Vary the solution density, 0.01 – 0.25 atoms/cm³, nominal = 0.10 atoms/cm³**
 - note that density variation ~ size variation (L)

Clustering vs Density (1,000 neutrons/cycle)

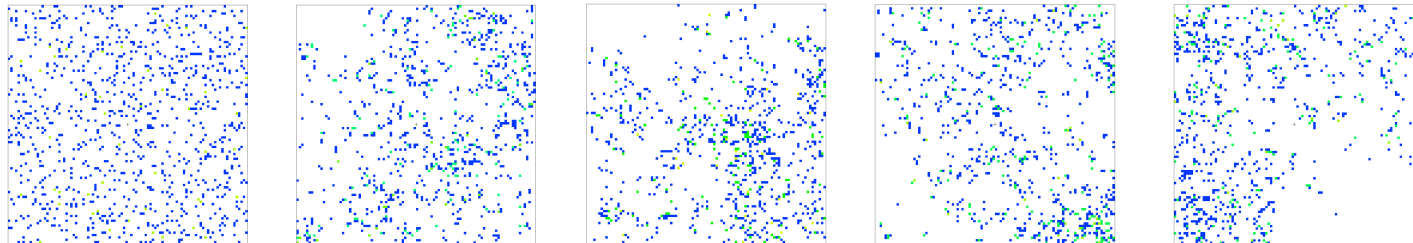
Density = 0.01

$\ell_F = 114.5$ cm
 $f_H^{\max} = 10608\%$
 $f_H = 55.2\%$
 $\ell_F/\ell_{\text{geom}} = 44.1\%$



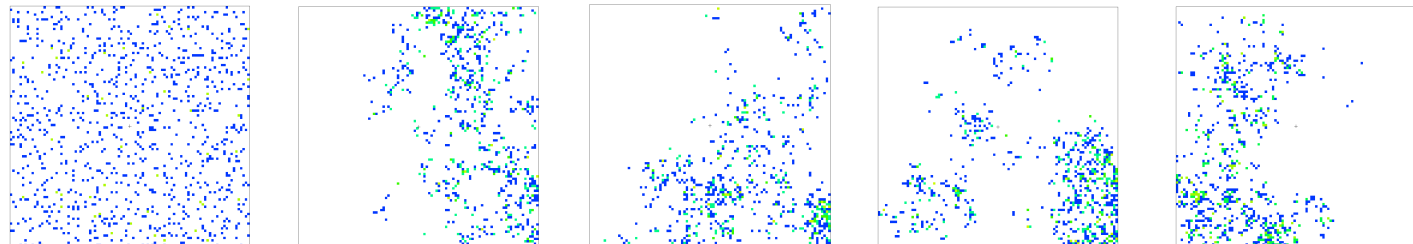
Density = 0.05

$\ell_F = 26.9$ cm
 $f_H^{\max} = 127\%$
 $f_H = 35.9\%$
 $\ell_F/\ell_{\text{geom}} = 10.1\%$



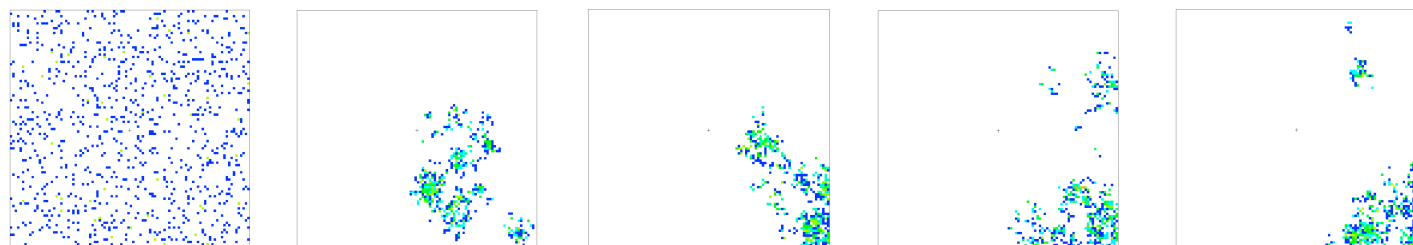
Density = 0.10

$\ell_F = 13.7$ cm
 $f_H^{\max} = 16.7\%$
 $f_H = 22.9\%$
 $\ell_F/\ell_{\text{geom}} = 5.1\%$



Density = 0.25

$\ell_F = 5.5$ cm
 $f_H^{\max} = 1.1\%$
 $f_H = 8.8\%$
 $\ell_F/\ell_{\text{geom}} = 2.1\%$



Cycle 1

Cycle 1000

Cycle 2000

Cycle 3000

Cycle 4000

A Real Problem

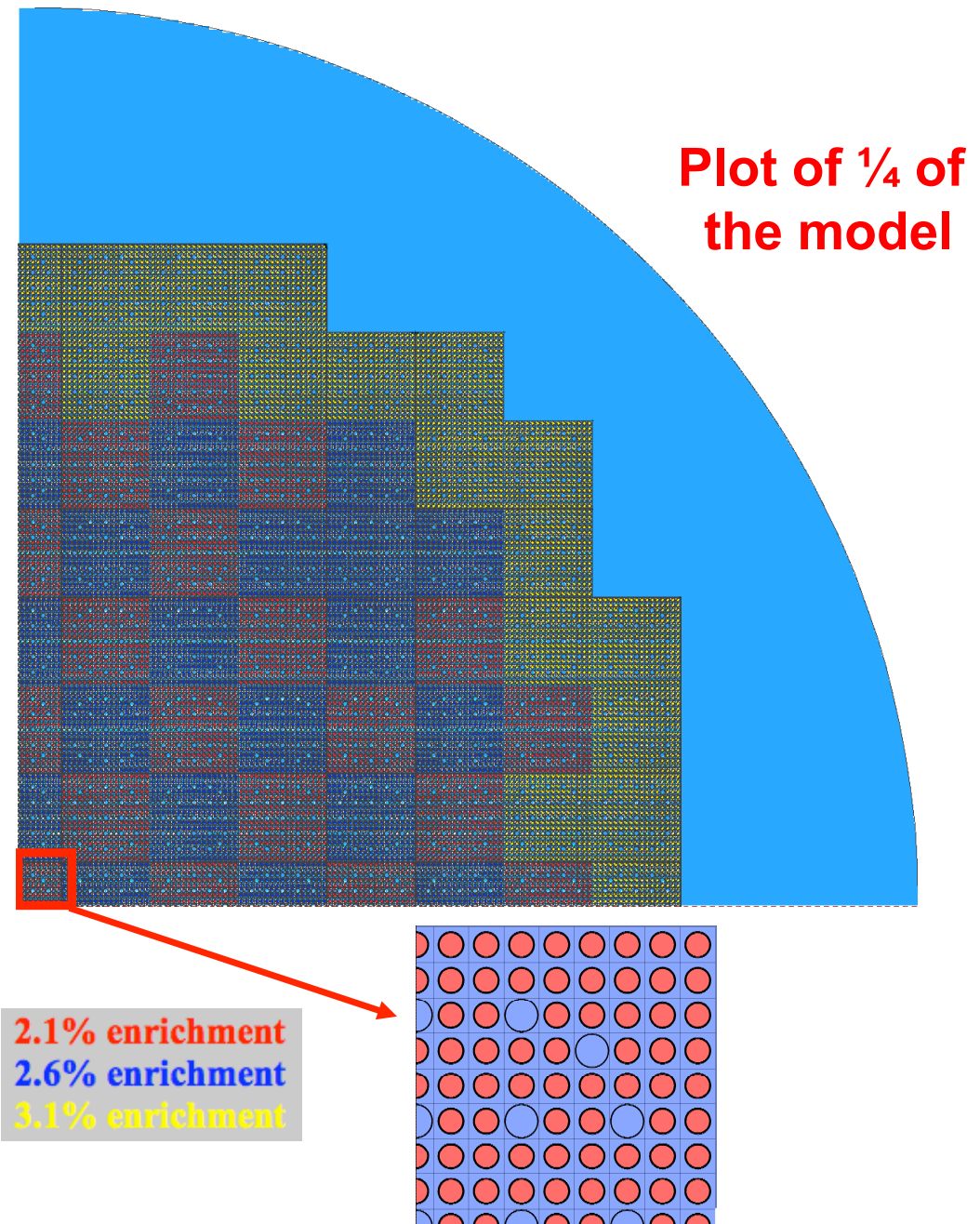
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PWR core

PWR2D – Realistic PWR Detailed Model

Nakagawa & Mori model of 2D PWR, realistic

- 50,952 fuel pins with cladding
- 4,825 water tubes for rods or detectors
- Each assembly:
 - Explicit fuel pins & rod channels
 - 17 x 17 lattice of pins in each assembly
 - Enrichments: 2.1%, 2.6%, 3.1%
- ENDF/B-VII.1 nuclear data
- Usually run with 100k neutrs/cycle
- For 3D whole-core, reactor was chosen to be 100 cm high, with water above & below



PWR2D – Clustering vs Neutrons/cycle

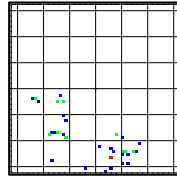
Whole-core,
with fuel in
100 cm axial,
324 x 324 x 100

Usually run
with 100k
neuts/cycle

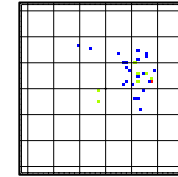
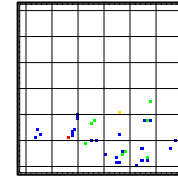
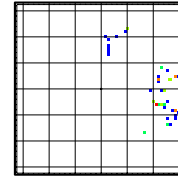
no clustering
in routine
calculations

$\ell_F = 19.1$ cm
 $f_{\max} = 14\%$
 $f_H = 1\%$

cycles to coalesce
to 1 chain = 65

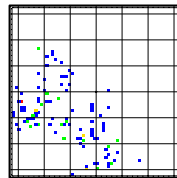


50 neutrons/cycle

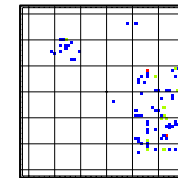
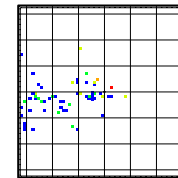
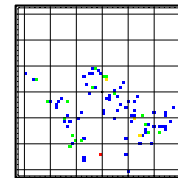


$\ell_F = 19.1$ cm
 $f_{\max} = 28\%$
 $f_H = 2\%$

cycles to coalesce
to 1 chain = 91

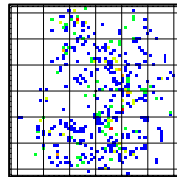


100 neutrons/cycle

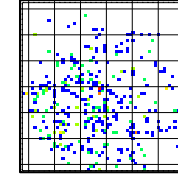
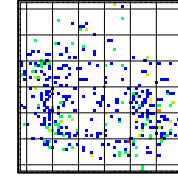
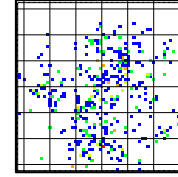


$\ell_F = 19.1$ cm
 $f_{\max} = 139\%$
 $f_H = 10\%$

cycles to coalesce
to 1 chain = 1061

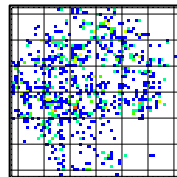


500 neutrons/cycle

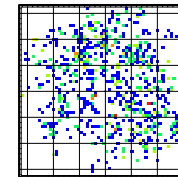
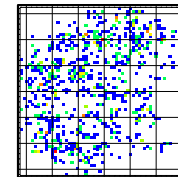
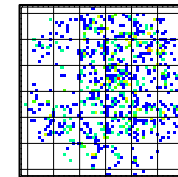


$\ell_F = 19.1$ cm
 $f_{\max} = 277\%$
 $f_H = 18\%$

cycles to coalesce
to 1 chain = 696

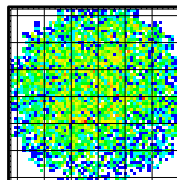


1,000 neutrons/cycle

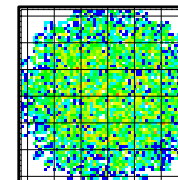
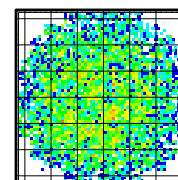
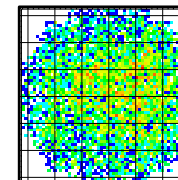


$\ell_F = 19.1$ cm
 $f_{\max} = 2775\%$
 $f_H = 74\%$

cycles to coalesce
to 1 chain = >> 4000



10,000 neutrons/cycle



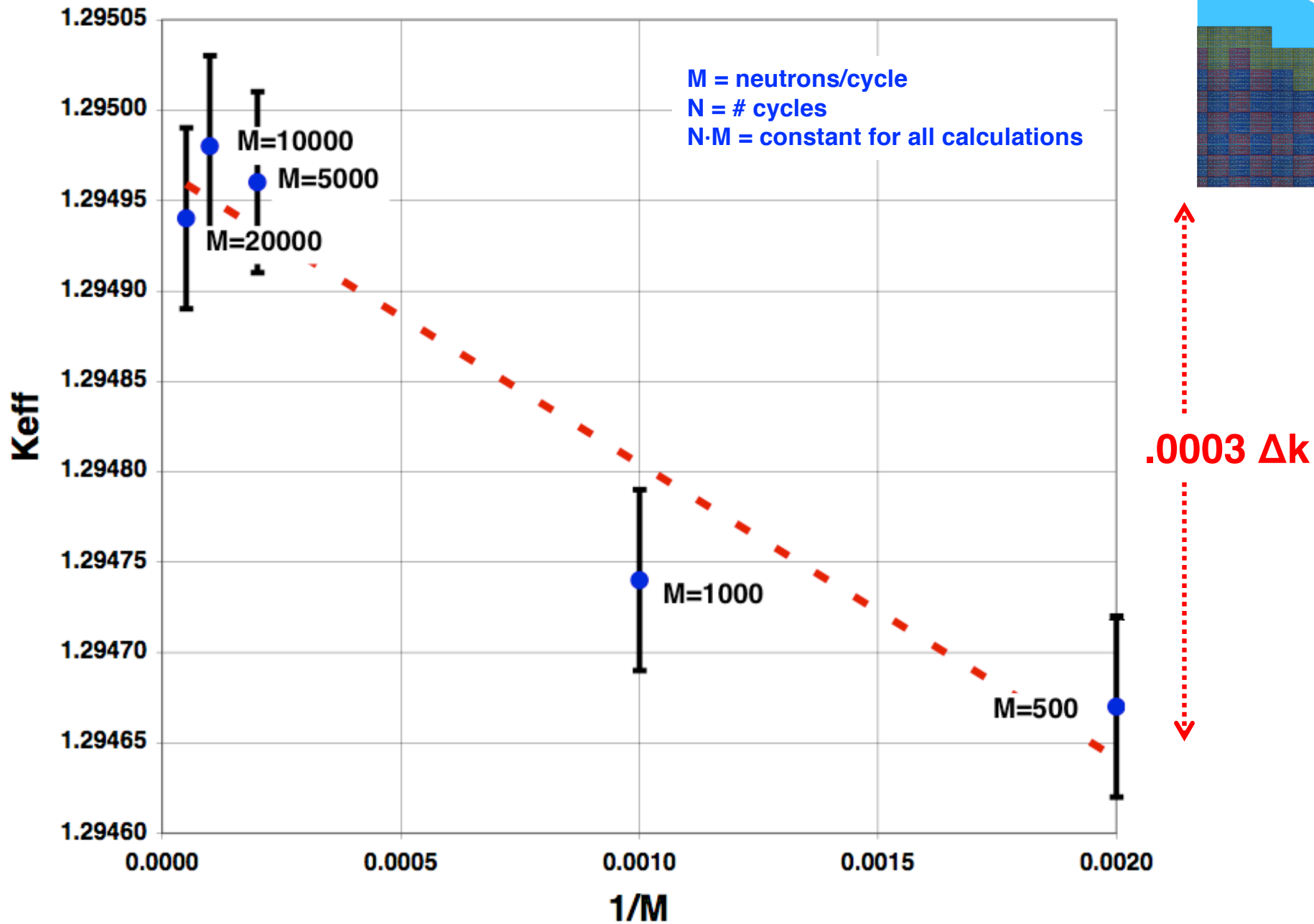
Cycle 1000

Cycle 2000

Cycle 3000

Cycle 4000

Bias in K_{eff} - for 2D $\frac{1}{4}$ -core, from LA-UR-09-05623



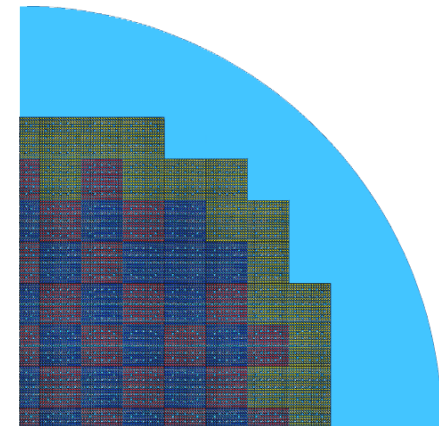
Bias in Tallies - for 2D $\frac{1}{4}$ -core, from LA-UR-09-05623

0.0	-0.5	-0.6	-0.2	-0.3	0.5	0.8									
-0.2	-0.7	-0.8	0.1	0.3	0.7	0.6									
-0.5	-0.7	-0.7	0.0	0.3	0.7	1.0	1.3	1.2	1.6	2.0					
-0.1	-0.7	-0.8	0.2	0.3	0.8	1.1	1.2	1.2	1.3	2.4					
-0.4	-0.6	-0.5	0.0	-0.1	0.2	0.7	0.6	1.4	2.0	1.9	2.7	3.2			
-0.7	-0.9	-0.8	-0.4	0.2	0.5	0.4	1.0	1.2	1.6	2.0	1.6	2.6			
-0.6	-0.3	-0.7	-0.6	-0.6	0.3	0.8	1.1	1.2	1.5	1.1	1.7	1.8			
-0.5	-0.8	-1.0	-0.8	-0.5	0.2	0.8	0.9	1.2	1.2	1.4	1.3	1.9			
-0.5	-0.9	-0.8	-1.0	-0.6	0.2	0.2	0.6	0.9	1.1	0.8	0.7	1.1	0.9	1.5	
-0.9	-0.9	-1.1	-1.0	-0.9	-0.1	0.2	0.6	0.8	0.6	0.6	0.6	1.3	1.2	1.1	
-1.2	-1.3	-1.2	-1.0	-0.6	-0.5	-0.3	0.2	0.9	0.7	1.1	0.9	1.3	1.2	1.1	
-1.3	-1.5	-1.0	-0.9	-0.7	-0.5	-0.6	0.3	0.4	0.5	1.3	1.4	2.1	1.9	1.6	
-1.7	-1.5	-1.1	-1.1	-0.6	-0.5	-0.2	-0.1	0.3	0.6	1.0	1.7	2.0	2.1	1.9	
-1.5	-1.5	-1.4	-1.0	-1.1	-0.8	0.0	0.1	0.3	0.4	1.0	1.0	1.5	3.1	2.3	
-1.6	-1.6	-1.2	-1.2	-0.6	-0.7	-0.4	-0.2	0.1	0.2	0.5	1.6	2.1	2.4	2.3	

Percent errors in
1/4-assembly fission rates
using 500 neutrons/cycle

Errors of -1.7% to +3.2%

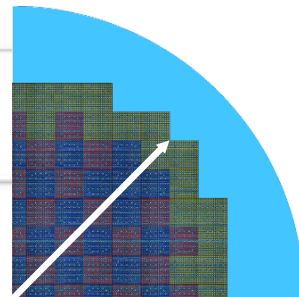
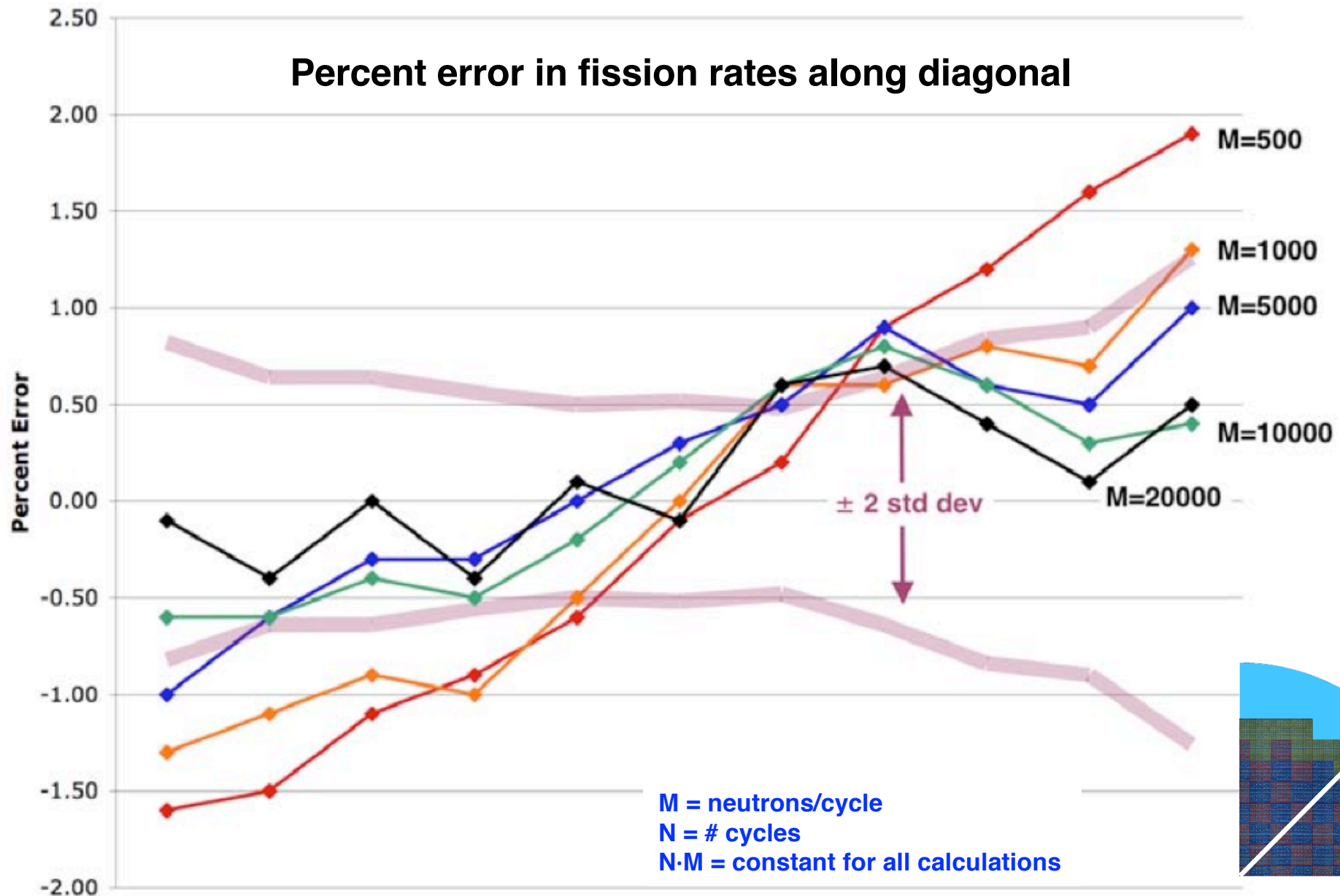
Statistics ~ .1% to .3%



Reference: ensemble-average of 25 independent calculations,
with 25 M neutrons each & 20K neutrons/cycle

Bias in Tallies - for 2D $\frac{1}{4}$ -core, from LA-UR-09-05623

Percent error in fission rates along diagonal

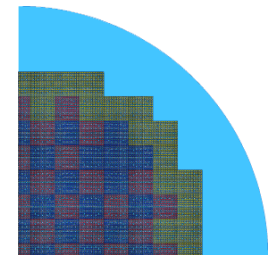


Bias in σ 's - for 2D 1/4-core, from LA-UR-09-05623

3.4	3.1	2.7	2.7	2.6	2.3	2.7								
3.3	3.7	3.6	3.7	3.7	2.7	2.9								
3.8	3.8	3.9	4.0	3.6	3.3	3.0	2.9	2.5	2.5	2.2				
3.8	3.9	4.2	3.3	3.5	3.4	3.2	3.6	3.0	3.0	2.8				
3.9	3.6	3.5	3.3	3.4	3.4	4.0	3.9	3.5	3.2	3.1	2.5	1.7		
4.1	3.8	3.5	3.2	2.9	2.6	2.9	3.2	3.1	2.8	2.7	1.9	1.7		
3.4	3.4	3.2	3.5	2.6	2.4	2.6	3.0	2.9	2.9	2.8	2.3	2.1		
4.2	3.5	3.4	3.1	2.7	2.3	2.0	2.4	2.5	2.5	2.1	2.3	2.3		
3.9	3.6	3.1	2.9	2.3	1.9	1.9	2.3	2.4	2.9	2.7	2.7	2.2	2.8	2.3
3.7	3.3	3.6	2.4	2.2	2.2	2.5	1.8	2.2	2.6	2.7	2.9	2.5	2.4	2.5
3.0	3.1	3.0	2.2	2.2	2.1	2.4	2.5	2.4	2.6	2.7	2.6	2.7	3.0	2.6
2.9	3.7	3.3	2.6	2.5	2.8	3.0	2.9	3.5	3.2	3.3	3.1	3.1	3.2	3.3
3.2	3.1	2.9	3.1	3.2	3.3	3.5	3.5	3.6	3.9	3.7	3.9	3.5	3.4	2.9
3.4	3.0	3.1	3.6	3.4	3.5	3.9	3.7	4.0	4.3	4.0	4.3	3.8	4.2	3.5
3.5	3.2	2.8	3.5	3.8	3.9	3.9	3.9	4.1	4.1	4.6	4.4	4.7	4.5	3.8

True relative errors in
1/4-assembly fission rates,
as multiples of calculated
relative errors, $\sigma_{\text{TRUE}} / \sigma_{\text{MCNP}}$

Calculated uncertainties
are 1.7 to 4.7 times smaller
than true uncertainties



Average factor = 3.1

Conclusions, Comments, Suggestions

Conclusions, Comments, Suggestions

- **For most practical problems, clustering is not a concern**
 - **Most problems today: 10k, 100k, or more neutrons/cycle**
 - mcnp6.2 will issue warning message if $< 10k$ neut/cycle
 - **For large reactors, it is routine to run very large neut/cycle, to get more efficient performance on parallel clusters**
- **For large solution tanks, clustering is a concern**
 - **Crit-safety practioners will probably not run 100k or 1M neut/cycle**
 - **There are some very, very large solution tanks (with very low Keff)**
 - **But fortunately, Keff result will be conservative, even with clustering**
 - Very large solution tank with clustering will be similar to infinite medium problem, with relatively few neutrons leaking. Keff will be overestimated, which is conservative for crit-safety
- **Very important to develop a diagnostic for clustering**
- **Cluster diagnostic for storage racks may be very different from large solution tanks (due to empty space, loose-coupling, etc.)**