The Intrinsic Source Constructor (ISC) and MCNPTools Libraries

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Slide 1



Outline

Intrinsic Source Constructor

Overview Using MISC to Generate Source Terms for MCNP ISC Library Usage

MCNPTools

Overview

Widgets

MCNPTools Accessor Classes

Mctal Class Meshtal (mesh-tally type B) Class Ptrac Class

Interactive MCNPTools Demo (time permitting)





- ISC is a library for generating intrinsic radiation sources for inputs into radiation transport codes
- ISC is written in C++ and bound to Python (via SWIG)
- ISC uses a CMake build system to build the C++ library and utilities and a Python setuptools setup.py to build the Python extensions
- ISC has with it a single binary utility MISC-the MCNP Intrinsic Source Constructor-that will generate SDEF distributions





- ISC has three classes of data files
 - 1. natural abundance files-contains naturally occurring abundances of isotopes
 - 2. radioactive decay data files-contains decay mechanisms, branching ratios, and daughter isotopes
 - 3. particle emission data files-contains the particle emissions from the decay of an isotope
- The data files are C++ Boost Serialized versions of ISC data storage classes
- The data files are typically stored in XML formats that can be interrogated in an editor (if one knows what they are looking for)





- to adequately capture decay channels, emissions, etc., ZA identifiers are insufficient
- ISC uses SZA identifiers where S is the long-lived isomeric state number (not necessarily the isomer's level number)

SZA = S * 1000000 + Z * 1000 + A

• For example, Pa-234m1 would be 1091234





- intended to facilitate usability
- allows a user to specify a natural ZA (i.e., A=000) and automatically expand it to isotopic ZAs
- Currently, the only available data set is the NIST natural abundances





- This file contains information about decay mechanisms, branching ratios, and decay daughters of SZAs
- Currently, these files do not include any information about spontaneous fission products
- Available data sets include data from ENDF/VI (not recommended) and ENDF/VII.1 radioactive decay sublibrary





Particle Emission Data Files

- For each SZA, a file exists that has the emissions by particle type
- The emission files do not contain any information about coincident emissions
- Beta-particle spectra are reconstructed based on Fermi theory from end-point energies
- Watt spectrum parameters exist for many/most spontaneously fissioning isotopes (taken from the SOURCES 4C) data set
- The currently available set of particle emission data are from the ENDF/VI (not recommended) and ENDF/VII.1 radioactive decay sublibraries (RDSL)





Why ENDF7.1 and not ENDF6

- vast improvements were made to the ENDF RDSL from ENDF/VI to ENDF/VII.0 and important corrections made in ENDF/VII.1
- below is a comparison of dose measurements to simulation from a DU sphere





- ISC has built into it a Bateman solver
- Materials compositions can be specified, aged, reset, and aged again if desired
- Emission spectra can be generated for either or both the original material composition and the aged composition





MISC Overview

- 1. MISC is a standalone binary utility that generates SDEF source distributions for MCNP
- 2. MISC reads and input file with entries generally of the form

 $\langle \text{keyword} \rangle = \langle \text{value} \rangle$

- 3. MISC has a User's Manual, but an example input is given herein
- 4. MISC generates source distributions for a single particle type at a time, but each distribution may have discrete and continuous components
- 5. MISC outputs two files and output file summarizing the run and a source file that contains the MCNP SDEF distributions





Example MISC Input File (Natural U)

```
# lines beginning with '#' are comments
                                                        matspec= 92000 -1
# specify output file
                                                         # specify the material density
output=test001.out
                                                         densitv=-18.0
#
                                                         #
# specify source distribution file
                                                         # specify the aging of the material in seconds
srcout=test001 src
                                                         age = 3.14e7
                                                         #
# specify the natural abundance file
                                                         # specify the particle type for the source
abundfile=nist.na.xml
                                                         particle=p
                                                         # specify vertical (v) or horizontal (h) formatted
# specify the decay data file
decayfile=endf7.dk.xml
                                                         # SDEF distributions
                                                         format=v
# specify the default particle emission library
pelib=endf7
                                                         # specify that electron sources should be converted
#
                                                         # to photon sources via a thick-target bremsstrahlung
# specify the material isotopic description with
                                                         # approximation
# SZAs (+/- = atom/mass fracs)
                                                         estobrems=y
```





Example MISC Output File

SZA	Isotope	Mass Frac
92000	U-Nat	1.00000e+00

Material density: 1.80000e+01 g/cm^3

The material was aged for 3.14000e+07 s using data from endf7.dk.xml

RESULTING MATERIAL SPECIFICATION:

SZA	Isotope	Atom Frac	Mass Frac
80206	Hg-206	1.27980e-33	1.10747e-33
81205	T1-205	1.51722e-47	1.30653e-47
81206	T1-206	4.29461e-32	3.71630e-32
81207	T1-207	2.37491e-23	2.06509e-23
81210	T1-210	2.18313e-28	1.92596e-28
82206	Pb-206	1.79148e-22	1.55023e-22
82207	Pb-207	5.40613e-19	4.70085e-19
82209	Pb-209	2.78302e-30	2.44339e-30
82210	Pb-210	9.45370e-20	8.33986e-20
82211	Pb-211	1.80241e-22	1.59765e-22
82214	Pb-214	2.14297e-23	1.92664e-23
83209	Bi-209	1.69762e-27	1.49045e-27
83210	Bi-210	5.50424e-23	4.85572e-23
83211	Bi-211	1.06845e-23	9.47069e-24
83214	Bi-214	1.59138e-23	1.43072e-23
83215	Bi-215	3.93728e-29	3.55639e-29
84210	Po-210	4.90580e-22	4.32777e-22
84211	Po-211	1.18508e-28	1.05045e-28
84214	Po-214	2.18935e-30	1.96829e-30
84215	Po-215	1.48237e-28	1.33895e-28
84218	Po-218	2.47808e-24	2.26965e-24



Atom density: 4.55401e-02 a/b*cm Mass density: 1.80000e+01 g/cm^3

TOTAL PARTICLE EMISSION RATES

```
Particle cm<sup>-3</sup> s<sup>-1</sup> g<sup>-1</sup> s<sup>-1</sup> Frac. Rate
n 2.42704e-01 1.34835e-02 3.80112e-07
p 1.86172e+05 1.03429e+04 2.91574e-01
a 4.52334e+05 2.51297e+04 7.08425e-01
Total 6.38506e+05 3.54726e+04 1.00000e+00
```





Example MISC Source File

- the source emission rate normalization is given per unit volume or mass of the source
- the source is broken into two pieces, the discrete and continuous spectra
- the source can also be output as a function of isotope if desired
- the starting source distribution number can be set in the input file

```
c wot should equal 1.86172e+05°VOL or 1.03429e+04°MASS
ci001 c
sp001 d 5.89603e-01
                     4 103970-01
           si002
     2 29873e-03 3 35218e-11
                  3 269366-21
     2 29892e-03
                  4.66760e-19
     2.29965e-03
                  2.08984e-14
     2 37789e-03 7 33374e-14
     3 094000+00 1 456460-14
                  4 599330-14
                 3 296196-15
                  1.8014
     3.18363e+00
                  5.09759e-14
           ci003
                        sn00?
     0.000000+00
                  0.000000+00
     2.00000e-02
                  2.89830e-01
                  1 373730-01
     6 000000-02
                  8 783250-02
     8.00000e-02
                  6.35485e-02
     1.00000e-01 4.92536e-02
     4.28000e+00 3.38511e-22
     4.30000e+00 1.56604e-22
     4 320000+00 5 995340-23
     4 340000+00 1 668120-23
     4.36000e+00 2.36227e-24
```





Interactive MISC example (Co-60)



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MISC Co-60 Example

```
# specify output file
                                                   matspec= 27060 1.0
output=co60.out
#
                                                    # specify the material density
# specify source distribution file
                                                    # normalize to 1 a/b-cm
srcout=co60.src
                                                    densitv=1.0
                                                    #
# specify the natural abundance file
                                                    # specify the aging of the material in seconds
abundfile=nist.na.xml
                                                    # 1 hl = 5.271 a = 1.663355e+8 s
                                                    age=1.663355e+8
# specify the decay data file
decavfile=endf7.dk.xml
                                                    # specify the particle type for the source
                                                    particle=p
#
# specify the default particle emission library
                                                    # specify vertical (v) or horizontal (h) formatted
pelib=endf7
                                                    # SDEF distributions
                                                    #
# specify the material isotopic description with
                                                    format=v
# SZAs (+/- = atom/mass fracs)
```





- MISC is useful...but the true power of ISC is that YOU can write your own scripts/programs to do whatever you want with decay data
- the ISC library supports C++ with Python bindings
- examples of using the library in Python follow





- this example (next slide) demonstrates how to load abundance, decay, and particle emission data files
- the natural abundance and decay data files load information for all SZAs
- the particle emission data is loaded on an SZA basis (loading all the data takes time)





ISC Example 1a: Loading Data Files Continued

import os
import isc # import the isc module

```
# set the path to the ISC data
iscdata = os.getenv("ISCDATA")
```

open an abundance data file and convert it to an abundance library abund_file = isc.AbundanceFile(os.path.join(iscdata,"nist.na.xml")) abund_lib = isc.AbundanceLib(abund_file)

open a decay data file and convert it to a decay data library decay_file = isc.DecayFile(os.path.join(iscdata,"endf7.dk.xml")) decay_lib = isc.DecayLib(decay_file)

open an emission file index (contains relative paths to emission data files) emission_index = isc.EmissionFileIndex(os.path.join(iscdata,"endf7.idx.xml")) # initialize an empty emission library emission_lib = isc.EmissionLib()

loop over all SZAs and import the emission data # NOTE: one need not load everything, only the things you need for sza in emission_index.GEZAs(): print("loading_emission_data_for_isotope_i:d)".format(sza)) emission_file = isc.EmissionFile(os.path.join(iscdata,emission_index.GetPath(sza))) emission_lib.SetFromEmissionFile(emission_file)





- for convenience and speed, sets of abundance, data, and particle emission data are bundled into binary data files
- these binary data files can be used to load sets of the data
- this is much faster than the previous example, but it loads all the emission data





ISC Example 1b: Loading Binary Data Files Continued

import os
import isc # import the isc module

set the path to the ISC data iscdata = os.getenv("ISCDATA")

initialize an empty abundance lib abund_lib = isc.AbundanceLib()

initialize an empty decay lib
decay_lib = isc.DecayLib()

initialize an empty emission library
emission_lib = isc.EmissionLib()

load the libraries from the prebuilt binary file isc.loadLibs(abund_lib, decay_lib, emission_lib, os.path.join(iscdata,"endf7.isc.bin"))





ISC Example 2: Querying Abundance Data

- this example (next slide) demonstrates how to get mass and natural abundance data out of the abundance library
- First, a set of all U isotopes the library knows about is generated
- Second, a loop is constructed over the set of all U isotopes and the mass and natural abundance is obtained
- Last, if the abundance is greater than zero, the SZA is added to the list of naturally occurring U isotopes





ISC Example 2: Querying Abundance Data Continued

import os
import isc # import the isc module

load the data
iscdata = os.getenv("ISCDATA")
abund.lib = isc.AbundanceLib()
decay.lib = isc.DecayLib()
emission_lib = isc.EmissionLib()
isc.loadLibs(abund_lib, decay_lib, emission_lib, os.path.join(iscdata,"endf7.isc.bin"))

get all U isotopes with data u.isos = abund_lib.GetIsosForZ(92) print(u.isos)

```
# get mass and abundance for each naturally occuring isotope
nat.u.isos = list()
for iso in u.isos:
    # get mass and abundance for iso
    mass = abund.lib.GetMass(iso)
    abundance = abund.lib.GetAbundance(iso)
    print(":7d1...;7.3f1...;12.5e!".format(iso, mass, abundance))
```

if abundances is non-zero add it to the list of naturally occurring isos
if(abundance > 0.0):
 nat.u.isos.append(iso)
print(nat.u.isos)





- this example shows how to query the decay data to obtain information such as half life, daughters, and branching ratios
- First, all the daughters of Cs-137 are requested
- Next, a loop over the number of decay pathways is constructed
- Then, for each decay pathway, the daughter and branching ratio is obtained





ISC Example 3: Querying Decay Data Continued

import os
import isc # import the isc module

load the data
iscdata = os.getenv("ISCDATA")
abund_lib = isc.AbundanceLib()
decay_lib = isc.DecayLib()
emission_lib = isc.EmissionLib()
isc.loadLibs(abund_lib, decay_lib, emission_lib, os.path.join(iscdata,"endf7.isc.bin"))

get all the daughters of Cs-137 cs137_daughters = decay_lib.GetAllDaughters(55137) print(cs137_daughters)

get the decay data for Cs-137 cs137_decay_data = decay_lib.GetDecayData(55137) print("Cs-137_half_life_=_{:12.5e}_s".format(cs137_decay_data.GetHalfLife()))

loop over the number of decay pathways
for i In range(cs137.decay.data.GetNumber()):
 # get daughter SZA is branching ratio
 daughter = cs137.decay.data.GetDaughter(i)
 branching_ratio = cs137.decay.data.GetBranchingRatio(i)
 print(":7d)_(:12.5e)".format(daughter, branching_ratio))





ISC Example 4: Querying Emission Data

- This example demonstrates how to get emission spectra of particular particle type out of the emission data
- First, all the spectra from Co-60 are requested
- Second, the gamma emission spectrum is obtained (an emission spectrum can have discrete and continuous pieces)
- Lastly, a loop over the discrete emissions is created and the energy and #/decay are printed





ISC Example 4: Querying Emission Data Continued

import os
import isc # import the isc module

load the data iscdata = os.getenv("ISCDATA") abund.lib = isc.AbundanceLib() decay_lib = isc.DecayLib() emission_lib = isc.EmissionLib() isc.loadLibs(abund_lib, decay_lib, emission_lib,\ os.path.join(iscdata,"endf7.isc.bin"))

get the emission spectra for Co-60 co60_spectra = emission_lib.GetSpectra(27060)

get a list of isc particle types for which spectra exist co60_particle_types = co60_spectra.GetParticleTypes() print(co60_particle_types)

loop over the particle types for ptype in co60.particle.types: if ptype == isc.ENDF.DECAY.GAMMA: print("Co-60.emits.gammas..ptype.=_.(:d)".format(ptype)) elif ptype == isc.ENDF.DECAY.BETAM: print("Co-60.emits.beta-..cptype.=_.(:d)".format(ptype)) elif ptype == isc.ENDF_DECAY.BETAP: print("Co-60_ensits_beta+_cptype_=_(:d)".format(ptype)) elif ptype == isc.ENDF_DECAY_IT: print("Co-60_has_internal_transition,_ptype_=_(:d)".format(ptype)) elif ptype == isc.ENDF_DECAY_ALPHA: print("Co-60_ensits_alptas,_ptype_=_(:d)".format(ptype)) elif ptype == isc.ENDF_DECAY_LECTRON: print("Co-60_ensits_electrons,_ptype_=_(:d)".format(ptype)) elif ptype == isc.ENDF_DECAY_XRAY: print("Co-60_ensits_arays,_ptype_=__(:d)".format(ptype))

get the gamma spectrum

co60_gammas = co60_spectra.GetSpectrum(isc.ENDF_DECAY_GAMMA) print("The_number_of_discrete_emissions_per_decay_is_{:.3f}".format(co60_gammas.GetDNorm()))

loop over all the discrete emissions print(":12s)...(:12s)".format("energy","#/decay")) for i in range(co60.gammas.GetDNumber()): # get the emission energy and probability/decay energy = co60.gammas.GetDIntensity(i) intensity = co60.gammas.GetDIntensity(i) print(":12.5e)...(:12.5e)".format(energy,intensity))





MCNPTools History and Overview

- MCNPTools was born out the continual need to process MCNP outputs
- MCNPTools is a library that provides object-oriented access to MCNP outputs
 - 1. MCTAL files
 - 2. MESHTAL B (MCNP5 style) files
 - 3. PTRAC files
 - 4. LNK3DNT files (not herein discussed)
- MCNPTools is is written in C++ and bound to Python and Perl
- MCNPTools also has some binary "widgets"





13dcoarsen Widget

USAGE: l3dcoarsen [--help] [--novoid] [--ifact ifact] [--jfact jfact] [--kfact kfact] [--maxmats maxmats] <LNK3DNT> [OUTPUT]

DESCRIPTION:

13dcoarsen coarsens a LNK3DNT file mesh by specified factors

OPTIONS:

- --help, -h : print usage infomation
- --novoid, -n : Make voids material '0' rather than the assumed material '1' (not recommended)
- --ifact, -i : Factor by which to coarsen in the first mesh dimension
- --jfact, -j : Factor by which to coarsen in the second mesh dimension (if applicable)
- --kfact, -k : Factor by which to coarsen in the third mesh dimension (if applicable)
- --maxmats, -m : Maximum umber of materials to keep include on the coarsened LNK3DNT file (default: same as original)
- LNK3DNT : LNK3DNT file name to coarsen
- OUTPUT : coarsened LNK3DNT output name (Default: lnk3dnt.coarse)

AUTHOR: Clell J. (CJ) Solomon [csolomon@lanl.gov]



Operated by Los Alamos National Security, LLC for the U.S. Department of Energy's NNSA



13dinfo Widget

USAGE: 13dinfo [--full] [--help] <LNK3DNT [LNK3DNT ...]>

DESCRIPTION:

13dinfo produces information about LNK3DNT files to stdout

OPTIONS:

full, -f	: Produce a full listing of the LNK3DNT contents (can greatly increase runtime)
help, -h	: print usage infomation
LNK3DNT	: LNK3DNT files about which to produce information

AUTHOR: Clell J. (CJ) Solomon [csolomon@lanl.gov]





13dscale Widget

USAGE: 13dscale [--help] <LNK3DNT> <FACTOR> [OUTPUT]

DESCRIPTION:

13dscale scales the dimensions of a LNK3DNT file

OPTIONS:

- --help, -h : print usage infomation
- LNK3DNT : LNK3DNT file to be scaled
- FACTOR : Scaling factor to be applied to the file
- OUTPUT : Output LNK3DNT file name [Default: LNK3DNT.scaled]

AUTHOR: Clell J. (CJ) Solomon [csolomon@lanl.gov]





mergemctals Widget

USAGE: mergemctals [--help] [--verbose] [--output output] <MCTAL [MCTAL ...]>

DESCRIPTION:

mergemctals statistically merges multiple MCNP MCTAL files into a single MCTAL file.

OPTIONS:

- --help, -h : print usage infomation
- --verbose, -v : Increase output verbosity
- --output, -o : Output MCTAL file name [Default: mergemctals.out]
- MCTAL : MCTAL file names to be merged
- AUTHOR: Clell J. (CJ) Solomon [csolomon@lanl.gov]

Note:

This is an alternative, not yet a replacement for, the Perl script that ships with MCNP. However, this binary can be compiled with MPI (requires C++ Boost Libraries), which speeds up merging.



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mergemeshtals Widget

DESCRIPTION:

mergemeshtals statistically merges multiple MCNP MESHTAL files into a single MESHTAL file.

OPTIONS:

help, -h	: print usage infomation
verbose, -v	: Increase output verbosity
output, -o	: Output MESHTAL file name [Default: mergemeshtals.out]
MESHTAL	: MESHTAL file names to be merged
AUTHOR: Clell J.	(CJ) Solomon [csolomon@lanl.gov]

Note:

This is an alternative, not yet a replacement for, the Perl script that ships with MCNP. However, this binary can be compiled with MPI (requires C++ Boost Libraries), which speeds up merging.

ATIONAL LABORATOR



mctal2rad Widget

USAGE: mctal2rad [--direct] [--help] [--log] [--transpose] <MCTAL>
 [TALLY [TALLY ...]]

DESCRIPTION:

mctal2rad converts an image tally from an MCNP MCTAL file into a TIFF image

OPTIONS:

direct, -d	: Produce an image of the direct contribution
help, -h	: print usage infomation
log, -l	: Produce an image of the \log of the <code>MCTAL</code> values
transpose, -t	: Transpose the image
MCTAL	: MCTAL file containing one or more image tallies
TALLY	: Tally number for which to produce the images

AUTHOR: Clell J. (CJ) Solomon [csolomon@lanl.gov]

NOTE:

This utility requires the TIFF libraries be installed.



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- The Mctal class stores information regarding mctal files
- Tally and Kcode data are stored in the MctalTally and MctalKcode classes, respectively
- Tally and Kcode data are only read into memory when requested via the getTally and getKcode methods of the Mctal class are called





general Mctal accessors GetCode() return the generating code name GetVersion() return the code version GetProbid() return the problem id number GetDump() return the corresponding runtpe dump number GetRandoms() return the number of histories GetTallyList() return a list of the tally numbers SummaryString() return a summary string of the mctal file

- GetTally(NUM) return a MctalTally object for tally NUM
- GetKcode() return a MctalKcode object





- general MctalTally accessors
 ID() return the tally id (number)
 SummaryString() return a summary string of the tally
- MctalTally bin accessors
 - GetFBins() return the facet bins
 - GetDBins() return the flagged/direct/collided bins
 - GetUBins() return the user bins
 - GetSBins() return the segment bins
 - GetMBins() return the multiplier bins
 - GetCBins() return the cosine bins
 - GetEBins() return the energy bins
 - GetTBins() return the time bins





- MctalTally value accessors
 - GetValue(F,D,U,S,M,C,E,T,PERT=NUM) return the tally value corresponding to the bin specified by the set of indices F, D, U, S, M, C, E, and T for perturbation NUM (defaults to unperturbed) [a negative value for any bin indicates use of the tally fluctuation chart index]
 - GetError(F,D,U,S,M,C,E,T,PERT=NUM) return the tally relative error corresponding to the bin specified by the set of indices F, D, U, S, M, C, E, and T for perturbation NUM (defaults to unperturbed) [a negative value for any bin indicates use of the tally fluctuation chart index]





general MctalKcode accessors

GetCycles()	-	return the number of total kcode cycles
GetSettle()	-	return the number of inactive kcode cycles
GetNdat()	-	return the number of data elements in a kcode entry
<pre>SummaryString()</pre>	-	return a summary string for the kcode data

MctalKcode data accessor

GetValue(VAL,CYCLE) – return the kcode value VAL for cycle CYCLE (defaults are averaged combined k_{eff} and last cycle). VAL is an integer between 0 and the number of data elements (see the manual for details)





- read tally 4 out of mctal file "my_mctal", extract the energy bin boundaries, store the tally values for each of the energy bins, and print the tally values for each of the energy bins
- 2. read the kcode data from mctal file "my_mctal" and print the k_{eff} value and error as a function of active cycle





mctal Example 1 – Python

```
from mcnptools import Mctal. MctalTallv
# construct the mctal class from mctal file "mv mctal"
m = Mctal("my_mctal")
tfc = MctalTallv.TFC: # alias for -1
# get tallv 4 from the mctal file
t4 = m.GetTallv(4):
# get the energy bins of tally 4
t4_e = t4.GetEBins();
# loop over energy bin indices to store and print tally bin value
# using the TFC bin for all other bins
# store the tally values with list comprehension
#fdusmcet
t4 evals = [ t4.GetValue(tfc.tfc.tfc.tfc.tfc.tfc.tfc) for i in range(len(t4 e)) ]:
# print the tally values
for i in range(len(t4_evals)):
```



print t4_evals[i];



mctal Example 2 – Python

```
from mcnptools import Mctal, MctalKcode
```

```
# construct the mctal class from the mctal file "my_mctal"
m = Mctal("my_mctal")
```

```
# get the kcode data
kc = m.GetKcode()
```

```
# alias for average combined keff
keff = MctalKcode.AVG_COMBINED_KEFF
# alias for average combined keff standard deviation
keff_std = MctalKcode.AVG_COMBINED_KEFF_STD
```

```
# loop over active cycles and print
for i in range(kc.GetSettle(),kc.GetCycles()):
    print i, "...", kc.GetValue(keff,i), "...", kc.GetValue(keff_std,i)
```







- The Meshtal class stores information about the mesh-tally type B data
- The MeshtalTally class stores information about a mesh tally
- The tally data is not read in until the getTally method of the Meshtal class is called





general Meshtal accessors

- return the generating code name
- return the code version
- GetProbid()

GetVersion()

GetComment()

SummaryString()

GetCode()

GetNps()

- return the problem id number
- return the problem comment
- return the number of histories
- return a summary string
- return a list of tallies in the file
- tally object accessor GetTally(NUM) -

GetTallyList()

M) – return a tally data object corresponding to tally NUM





MeshtalTally Class Accessors I

general MeshtalTally accessors

- ID()
- GetXRBounds()
- GetYZBounds()
- GetZTBounds()
- GetEBounds()
- GetTBounds()
- GetXRBins()
- GetYZBins()
- GetZTBins()
- GetEBins()
- GetTBins()

- return the tally id (number)
- _ return the x/r bin boundaries
- return the y/z bin boundaries _
- return the z/theta bin boundaries
- return the energy bin boundaries
- return the time bin boundaries
- return the x/r bin centers
- return the v/z bin centers _
- return the z/theta bin centers _
- return the energy bins _
- return the time bins _
- GetVolume(I,J,K) return the volume of specified element





MeshtalTally value accessors

- GetValue(I, J, K, E, T) return the tally value for (I, J, K)th element for energy bin E and time bin T. E and T default to the totals if left unspecified.
- GetError(I, J, K, E, T) return the tally relative error for the (I,J,K)th element for energy bin E and time bin T. E and T default to the totals if left unspecified.





1. read in a meshtal file, extract tally 4, and print the 2-D slice for the z index of 5





meshtal Example 1 – Python

```
from mcnptools import Meshtal, MeshtalTally
from sys import stdout
```

```
# construct the meshtal class from meshtal file "my_meshtal"
m = Meshtal("my_meshtal")
```

```
# get tally 4 from the meshtal file
t4 = m.GetTally(4)
```

```
# get the x and y bin centers
x = t4.GetXRBins()
y = t4.GetYZBins()
```

```
# loop over x and y bins indices and print the tally value for
# z index of 5
for i in range(len(x)):
    for j in range(len(y)):
        stdout.write("{:12.5e}".format(t4.GetValue(i,j,5)))
        stdout.write("\n")
```





Outline

ntrinsic Source Constructor Overview Using MISC to Generate Source Terms for MCNP ISC Library Usage

MCNPTools Overview Widgets MCNPTools Accessor Classes Mctal Class Meshtal (mesh-tally type B) Class Ptrac Class Interactive MCNPTools Demo (time permitting)





- The Ptrac class manages data for MCNP's ptrac files and can handle BOTH text and binary versions (assumes binary as default)
- The PtracHistory class manages data for an individual history
- The PtracNps class handles the starting information for a history
- the PtracEvent class handles information for particle events





 ReadHistories(NUM) – reads NUM histories and all associated events into memory as a list; allows "chunk" processing of a file without filling memory





- GetNPS() return a PtracNps class to the history (NPS) information
- GetNumEvents() return the number of events recorded for the history
- GetEvent(NUM) return a PtracEvent class to the history's NUMth event





- NPS() return the history number
- Cell() return the filtering cell from CELL keyword (if present)
- Surface() return the filtering surface from SURFACE keyword (if present)
- Surface() return the filtering tally from TALLY keyword (if present)
- Value() return the tally score from TALLY keyword (if present)





- Type() return the event type
- Has(DATA) return true if the event has data type DATA
- Get(DATA) return the value of the data type DATA





- Open a ptrac file named "my_ptrac", read in histories, and print the x, y, and z location and energy of all bank events (this is when particles are pulled from the bank)
- Open at ptrac file named "my_ptrac", read in histories, and, for all surface crossings, print the x, y, and z location and the angle (in degrees) of the crossing with respect to the surface normal





ptrac Example 1 – Python

from mcnptools import Ptrac
from sys import stdout

```
# explicitly open the file as a binary ptrac
p = Ptrac("my_ptrac", Ptrac.BIN_PTRAC)
```

```
# initialize counter
cnt = 0
# read histories in batches of 10000
hists = p.ReadHistories(10000)
while hists:
```

```
# loop over all histories
for h in hists:
    # loop over all events in the history
    for e in range(h.GetNumEvents()):
    event = h.GetEvent(e)
    if event.Type() == Ptrac.BNK:
        cnt += 1
    stdout.write("{:13d}{:13.5e}{:13.5e}{:13.5e}{:13.5e}\n".format( \
        cnt,
        event.Get(Ptrac.X), \
        event.Get(Ptrac.Z), \
        etert.Get(Ptrac.Z), \
        event.Get(Ptrac.Z), \
        etert.Get(Ptrac.Z), \
        etert.Get(Ptrac.Z), \
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        event.Get(Ptrac.Z), \
        etert.Get(Ptrac.Z), \
        event.Get(Ptrac.Z), \
        event.Get(P
```

```
hists = p.ReadHistories(10000)
```





ptrac Example 2 – Python

from mcnptools import Ptrac
from sys import stdout

```
# explicitly open the file as a binary ptrac
p = Ptrac("my_ptrac", Ptrac.BIN_PTRAC)
```

read histories in batches of 10000
hists = p.ReadHistories(10000)

while hists:

```
# loop over all histories
for h in hists:
    # loop over all events in the history
for e in range(h.GetNumEvents()):
    event = h.GetEvent(e)
    if event.Type() == Ptrac.SUR:
        stdout.write("{:13.5e}{:13.5e}{:13.5e}{:13.5e}\n".format( \
        event.Get(Ptrac.X), \
        event.Get(Ptrac.Z), \
        event.Get(Ptrac.ANGLE) \
        )
    )

    Thists = p.ReadHistories(10000)
    Los Alamos
```





Interactive MCNPTools Demo



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