

MCNP-WHISPER Methodology for Nuclear Criticality Safety Validation

This work supports:

US DOE Nuclear Criticality Safety Program

US DOE Stockpile Stewardship Program

LANL Nuclear Criticality Safety Division

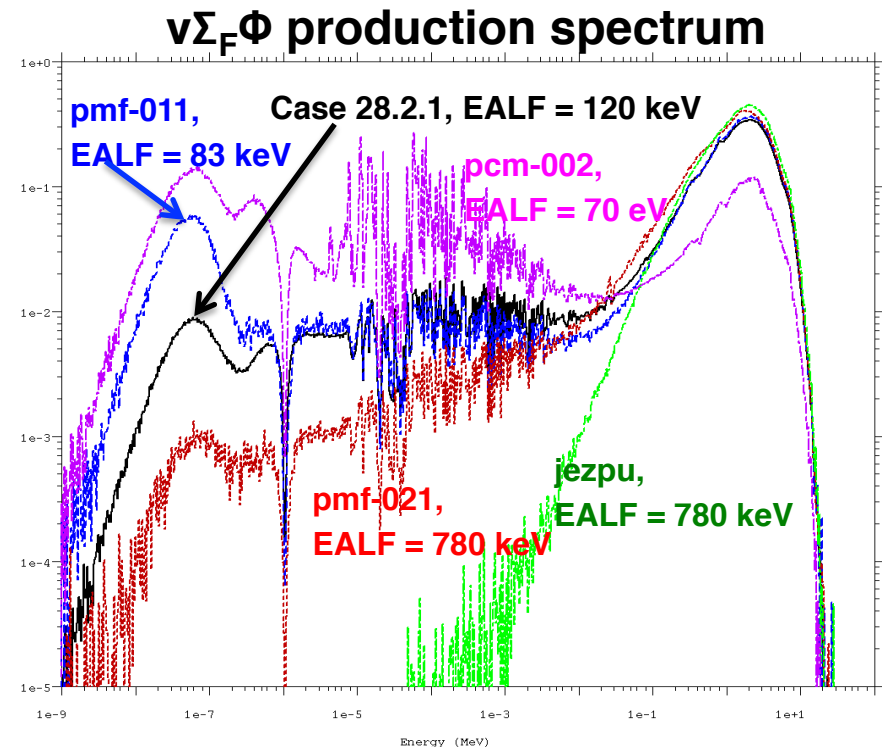
LANL PF4 Restart

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- Nuclear Criticality Safety requires validation of computational methods
- Validation involves comparing calculation vs experiment for many benchmarks similar to the application of interest
- Neutron spectra are complex functions of geometry, materials, nuclear cross-sections, etc.
- Simple metrics cannot capture the complexity of a fissile system
- During the past 20 years, a powerful set of tools has been developed based on sensitivity-uncertainty methods



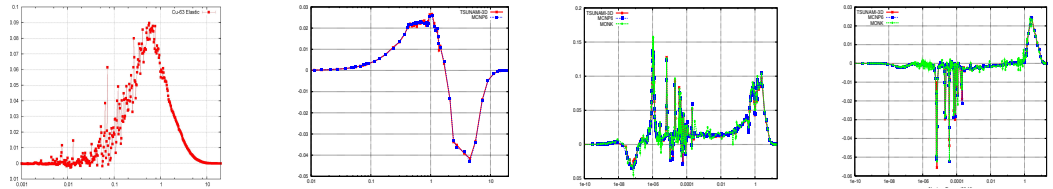
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- **MCNP** determines **sensitivity profiles** to characterize the neutronics of an application or benchmark, $S(\text{energy, reaction, isotope})$, $S = (dk/k) / (d\sigma/\sigma)$
- **WHISPER** uses sensitivity profiles & data covariances to select similar benchmarks, determine bias, bias-uncertainty, & margin-of-subcriticality for setting the **Upper-Subcritical-Limit (USL)**

- The **sensitivity coefficient** is the ratio of relative change in k-effective to relative change in a system parameter:

$$S_{k,x} = \frac{dk/k}{dx/x} = - \frac{\langle \psi^\dagger, (\Sigma_x - S_x - k^{-1}F_x) \psi \rangle}{\langle \psi^\dagger, k^{-1}F \psi \rangle}$$

- $S_{k,x}(E)$ is the **sensitivity profile**, that includes all isotopes, reactions, & energies for a system:



etc.

- MCNP Monte Carlo** uses the Iterated Fission Probability method to compute adjoint-weighted integrals for the sensitivity profiles
 - Tally scores are collected in original generation, adjoint-weighting is based on the progeny in the asymptotic generation

