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**Pages 223-282 only:**

**Advanced Tallies**

- PTRAC**
- FM, F8, FTn (CAP, PHL, TAG)**
- F5, FY5, mesh, radiography**



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# Advanced Tallies

# Additional Tally Capabilities

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- Particle Tracks (**PTRAC**)
- Reaction Multipliers (**FM**)
- Pulse Height (**F8**)
- Special Treatments (**FTn**)
  - Neutron Capture (**CAP**)
  - Pulse-Height Light (**PHL**)
  - Tally Tagging (**TAG**)
- Next Event: Point (**F5**), Ring (**FY5**)
- Special Tallies
  - Mesh Tallies
  - Radiography

# PTRAC – Particle Tracks

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**PTRAC enables writing particles to a file for post-processing**

Useful for particle track plotting in MORITZ

Useful for post-processing using various filters

**PTRAC differs from SSW / SSR:**

Filters events

Can be ascii

**PTRAC format:**

**PTRAC KEYWORD=value(s) ...**

***Example:***

**PTRAC FILE=asc WRITE=all EVENT=sur  
MAX=50000**

# PTRAC – Particle Tracks

---

## PTRAC Options:

FILE = <b>asc</b> or <b>bin</b>	(default=bin)
MAX = maximum number of events	(default=10000)
WRITE = <b>pos</b> or <b>all</b>	pos=x,y,z (default) all=x,y,z,u,v,w,E,W,T
EVENT = <b>src, bnk, col, sur, ter, cap</b>	(default= <b>all</b> )
TYPE = <b>p, p, p, ...</b>	particle types n, p, ...
FILTER = <b>values,parameter ...</b>	
= <b>2,ICL</b>	(cell 2)
= <b>.001,14.0,E</b>	(.001 < E < 14.0)

## History Filter Keywords:

**NPS, CELL, SURFACE, TALLY, VALUE**

*... only write PTRAC events to particles in NPS range, passing through cells*

*in CELL list, crossing surfaces in SURFACE list, contributing to tallies in TALLY list, etc.*

# Exercise 1

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**copy c:\MCNP6\EXAMPLES\ata1**

**Check that the following line is present**

***PTRAC FILE=asc WRITE=all***

**Run the problem and examine output.**

**mcnp6 i=ta1**

# Exercise 1 - PTRAC output

See Appendix F in MCNP6 Manual

```
-1
mcnp      6                01/15/14 02/03/14 11:50:33
tal3 - PTRAC Example
 1.4000E+01  1.0000E+00  1.0000E+02  0.0000E+00  0.0000E+00  1.0000E+00  1.0000E+00  0.0000E+00  1.0000E+00  1.0000E+04
 0.0000E+00  0.0000E+00  0.0000E+00  0.0000E+00  0.0000E+00  0.0000E+00  1.0000E+00  2.0000E+00  0.0000E+00  0.0000E+00
 2      7      9      8      9      8      9      8      9      8      9      0      4      0      0      0      0      0      0      0
 1      2      7      8      9      16      17      18      19      20      21      22      23      24      25      26      27      28      7      8      10      11      16      17      18      19      20      21      22      23
24      25      26      27      28      7      8      12      13      16      17      18      19      20      21      22      23      24      25      26      27      28      7      8      10      11      16      17      18      19
20      21      22      23      24      25      26      27      28      7      8      14      15      16      17      18      19      20      21      22      23      24      25      26      27      28
      1      1000
      3000      1      40      2      3      0      0
-0.20000E+01  0.00000E+00  0.00000E+00  0.10000E+01  0.00000E+00  0.00000E+00  0.00000E+00  0.20000E+01  0.10000E+01  0.00000E+00
      3000      2      2.2      179      2      2      2      0
-0.11000E+01  0.00000E+00  0.00000E+00  0.10000E+01  0.00000E+00  0.00000E+00  0.00000E+00  0.20000E+01  0.10000E+01  0.30021E-02
      4000      3      1.2      179      2      1      1      0
-0.10000E+01  0.00000E+00  0.00000E+00  0.10000E+01  0.00000E+00  0.00000E+00  0.00000E+00  0.20000E+01  0.10000E+01  0.33356E-02
      4000      3      3000      -1      2      1      1      1
-0.31983E+00  0.00000E+00  0.00000E+00  -0.92576E-01  0.94706E+00  -0.30743E+00  0.37435E+00  0.10000E+01  0.56044E-02
      4000      3      3000      -1      2      1      1      2
-0.34593E+00  0.26700E+00  -0.86672E-01  -0.48678E+00  -0.51751E+00  0.70372E+00  0.17852E+00  0.10000E+01  0.65448E-02
      5000      3      3000      -3      2      1      1      3
-0.47352E+00  0.13135E+00  0.97781E-01  -0.48678E+00  -0.51751E+00  0.70372E+00  0.17852E+00  0.10000E+01  0.74191E-02
      2011      3      12      1      2      1      1      3
-0.47352E+00  0.13135E+00  0.97781E-01  -0.48678E+00  -0.51751E+00  0.70372E+00  0.17852E+00  0.10000E+01  0.74191E-02
. . .
```

Header

Particle track information

# Exercise 1 - PTRAC output

```
-1
mcnp      6
tal3 - PTRAC Example
```

```
01/15/14 02/03/14 11:50:33
```

} Code Version, Run ID and Title

**BUFFER=100**

**FILE=ASC**

**MAX=10000**

} PTRAC settings echoed

```
1.4000E+01 1.0000E+00 1.0000E+02 0.0000E+00 0.0000E+00 1.0000E+00 1.0000E+00 0.0000E+00 1.0000E+00 1.0000E+04
0.0000E+00 0.0000E+00 0.0000E+00 0.0000E+00 0.0000E+00 0.0000E+00 1.0000E+00 2.0000E+00 0.0000E+00 0.0000E+00
```

**WRITE=ALL**

**Number of variables listed on given lines**

```
2 7 9 8 9 8 9 8 9 8 9 0 4 0 0 0 0 0 0 0
```

**Types of variables listed on given lines**

```
nps          src          bnk
1 2 | 7 8 9 16 17 18 19 20 21 22 23 24 25 26 27 28 | 7 8 10 11 16 17 18 19 20 21 22 23
24 25 26 27 28 | 7 8 12 13 16 17 18 19 20 21 22 23 24 25 26 27 28 | 7 8 10 11 16 17 18 19
20 21 22 23 24 25 26 27 28 | 7 8 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28
                                sur
                                col
                                ter
```



# Exercise 1 - PTRAC output

See Appendix F in MCNP6 Manual

1	Event node	nsr	particle	cell	material	ncp				
	<b>x</b>	<b>y</b>	<b>z</b>	<b>u</b>	<b>v</b>	<b>w</b>	<b>energy</b>	<b>weight</b>	<b>time</b>	
-0	Event node	surface angle	particle	cell	material	ncp	0000E+01	0.10000E+01	0.00000E+00	
	3000	2	2.2	179	2	2	2	0		sur Line (7 8 12 13 16 17 18 19) (20 21 22 23 24 25 26 27 28)
	-0.11000E+01	0.00000E+00	0.00000E+00	0.10000E+01	0.00000E+00	0.00000E+00	0.20000E+01	0.10000E+01	0.30021E-02	
	4000	3	1.2	179	2	1	1	0		
-0	Event node	ZA	MTP	particle	cell	material	ncp	01	0.10000E+01	0.33356E-02
	4000	3	3000	-1	2	1	1	1		col Line (7 8 10 11 16 17 18 19) (20 21 22 23 24 25 26 27 28)
	-0.31983E+00	0.00000E+00	0.00000E+00	-0.92576E-01	0.94706E+00	-0.30743E+00	0.37435E+00	0.10000E+01	0.56044E-02	
.	Event node	nter	branch	particle	cell	material	ncp			ter Line (7 8 14 15 16 17 18 19) (20 12 22 23 24 25 26 27 28)
.	Event node	ZA	NTYN	particle	cell	material	ncp	+00	0.10000E+01	0.74191E-02
	5000	4	32000	3	3	1	1	3		bnk Line (7 8 10 11 16 17 18 19) (20 21 22 23 24 25 26 27 28)
	-0.47352E+00	0.13135E+00	0.97781E-01	-0.18214E+00	0.77149E-01	0.98024E+00	0.17852E+00	0.10000E+01	0.74191E-02	

# Reaction Multipliers (FM)

**Form: FMn (C<sub>1</sub> m<sub>1</sub> R<sub>1</sub>) (C<sub>2</sub> m<sub>2</sub> R<sub>2</sub>) . . . T**

n = tally number

C<sub>i</sub> = multiplicative constant (if -1 for n=4, use cell ρ<sub>a</sub>)

m<sub>i</sub> = material number identified on an Mm card

R<sub>i</sub> = a combination of ENDF reaction numbers

**What It Does:**

**Common Neutron R Values**

$$C \cdot \int \Phi(E) R_m(E) dE$$

-1 = total cross section

-2 = absorption

-4 = heating (MeV/collision)

-6 = fission cross-section

-7 = fission ν

-8 = fission Q (MeV/fission)

# FM Reaction Values

NEUTRONS		PHOTONS		PROTONS	
1	Total	-1	incoherent	1	total
-2	Absorption	-2	coherent	2	non-elastic
-4	Heating	-3	photoelectric	3	elastic
-5	gamma prod'n	-4	pair production	4	heating
-6	total fission	-5	total	>4	other rxns.
-7	fission $\nu$	-6	heating	100R	particle I
-8	fission Q	1	PN total		from rxn. R
16	(n,2n)	2	PN non-elastic		
17	(n,3n)	3	PN elastic		
18	(n,fx)	4	PN heating		
		> 4	PN other rxns.		
		100R	PN particle I from rxn. R		

# Examples of FM

---

**F2:N** 1 2 \$ 36 tally bins

**FM2** (1.0) (2.0) (3.0) \$ Constant multipliers

**E2** .5 1 2 4 10 T \$ Energy bins

**F4:N** (1 2) 3 T \$ 6 tally bins

**FM4** (-1 1 -6 -7) \$ Track-length estimate of  $k_{\text{eff}}$

(-1 2 1 -4) \$ Neutron Heating (MeV/cm<sup>3</sup>)

# Pulse Height Tally

---

$$F8: \langle p \rangle C_1 \dots C_n$$

- **Different from all other tallies**
  - Surface estimator of cell energy deposition
  - Can use variance reduction with F8
  - Energy is accumulated from all tracks of a particle's history
- **Mimics pulse-height detectors: energy bins contain pulses**
  - Energy  $< 0$ : non-analog negative score balance
  - Energy  $\sim 0$ : particles pass through without energy loss
  - Energy  $> 0$ : pulse of W put into appropriate energy bin

# Exercise 2: Pulse Height Tallies

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**copy c:\MCNP6\EXAMPLES\atal2**

- **Use energy bins 0 1.e-6 0.1 199i 2.0**
- **Do a pulse-height tally (F8) in H2O**
- **Run the problem.**

**mcnp6 i=tal2 n=tal2a.**

- **Examine output file summary table.**
- **Plot tally 8 results.**

# Exercise 2: Pulse Height Tallies

---

- Change radius from 10000 to 10 cm
- Run the problem.

`mcnp6 i=tal2 n=tal2b.`

- Examine output file summary table and Plot tally 8.
- What is different and why?
- Repeat the last two steps with “mode p e” and F8:e instead of F8:p.

# Exercise 2: Pulse Height Tallies

---

**copy c:\MCNP6\EXAMPLES\atal2**

- **Use energy bins 0 1.e-6 0.1 199i 2.0**
- **Do a pulse-height tally (F8) in H2O**
- **Do an energy-deposition pulse-height tally (\*F8)**
- **Do energy deposition (F6) and equivalent FM4 energy deposition**
- **Do +F6 energy deposition**
- **Plot the tallies**



# Exercise 3: Pulse Height Tallies

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**copy c:\MCNP6\EXAMPLES\atal3**

- **Use energy bins 0 1.e-6 .001 .01 .1 100ilog 101**
- **Do a pulse-height tally (F8) in H2O**
- **Do an energy-deposition pulse-height tally (\*F8)**
- **Do energy deposition (F6) and equivalent FM4 energy deposition**
- **Do +F6 energy deposition**
- **Plot the tallies**

# Tally Treatments (FT)

**Form:**  $FT_n \text{ id}_1 \text{ p}_{1,1} \text{ p}_{1,2} \dots \text{ id}_2 \text{ p}_{2,1} \text{ p}_{2,2} \dots \dots$

$n$  = tally number

$\text{id}$  = Special tally treatments given below

$\text{p}_{i,j}$  = parameter  $j$  for the  $i^{\text{th}}$  tally treatment.

## Special tally treatments:

**FRV** Fixed arbitrary reference direction for tally 1 cosine binning.

**GEB** Gaussian energy broadening.

**TMC** Time convolution.

**INC** Identify the number of collisions.

**ICD** Identify the cell from which each detector score is made.

**SCX** Identify the sampled index of a specified source distribution.

**SCD** Identify which of the specified source distributions was used.

**ELC** Electron current tally.

**PTT** Put different multigroup particle types in different user bins.

**PHL** Pulse-height light tally with anticoincidence (f8 only). **(MCNP6)**

**CAP** Coincidence capture (f8 only). **(MCNP6)**

**RES** Residual nuclei. **(MCNP6)**

**TAG** Tally tagging. **(MCNP6)**

**LET** Tally stopping powers instead of energy. **(MCNP6)**

**ROC** Receiver-operator characterization **(MCNP6)**

# FTn TAG : Tally Tagging

---

**Tally tagging separates a tally into bins by how and where the scoring particle was produced:**

- 1) a cell of interest where particles are produced;**
- 2) a target nuclide from which the particle is emitted; and**
- 3) a reaction or, in the case of spallation, a residual nuclide of interest.**

**not for F8 tallies!**

## FTn TAG a

**a=1 : collided particles lose their tag; bremsstrahlung and annihilation photons included in the bin of collided particles;**

**a=2 : collided particles lose their tag; bremsstrahlung and annihilation photons given special tags for segregation;**

**a=3 : all collided particles retain their production tag.**

# FTn TAG : Tally Tagging

---

*FUn card required:*

*FUn bin<sub>1</sub> bin<sub>2</sub> ... bin<sub>N</sub>*

*bin<sub>j</sub> = CCCCCZZAAA.RRRRR*

**CCCCC** = *cell number or 00000*

**ZZAAA** = *target nuclide identifier*

**RRRRR** = *reaction identifier (e.g. 00102 for n,γ) or residual nuclide ZAID for model reactions*

# FTn TAG : Tally Tagging

---

$bin_j = \text{CCCCZZAAA.RRRRR}$  special cases:

- 0000000001** or **-1** source particle tag for all cells
- CCCC00001** source (i.e., uncollided) particle tag for cell CCCC
- 0000000000** or **0** scattered particle tag
- 1000000000** or **1e10** everything else tag

Photon tally special designations for ZZAAA.RRRRR:

- 00000.00001** bremsstrahlung from electrons
- ZZ000.00003** fluorescence from nuclide ZZ
- 00000.00003** K x-rays from electrons
- 00000.00004** annihilation photons from e-
- ZZ000.00005** Compton photons from nuclide ZZ
- ZZAAA.00006** muonic x-rays from nuclide ZZAAA

# FTn TAG : Tally Tagging

---

*binJ* = **CCCCZZAAA**.RRRRR *special cases:*

## Electron special designations for ZZAAA.RRRRR:

- ZZ000.00001** *photoelectric from nuclide ZZ*
- ZZ000.00003** *Compton recoil from nuclide ZZ*
- ZZ000.00004** *pair production from nuclide ZZ*
- ZZ000.00005** *Auger electron from nuclide ZZ*
- 00000.00005** *Auger electron from electrons*
- 00000.00006** *knock-on electrons*

## Neutron/photon special designations for ZZAAA.RRRRR:

**ZZAAA.99999** *delayed particles from fission or residuals  
of ZZAAA*

# FTn TAG : Tally Tagging

## Examples:

F5:P 0 0 0 1

FT5 TAG 3

FU5 -1.0 0000106012.00005 0000106012.00000 0000026056.00102  
0000026056.00000 0000000000.00051 10000000000.00000

## **-1.0 Source photons**

0000106012.00005

*Compton from 12C cell 1*

0000106012.00000

*Remaining photons from 12C in cell 1*

0000126056.00102

*Capture gammas from 56Fe in cell 1*

0000026056.00000

*Remaining photons/gammas from 56Fe*

0000000000.00051

*Remaining 1st inelastic level [n,n'] gammas*

10000000000.00000

*Remaining gammas*

*Physics muon example will use tagging*

# Exercise 7: Tally Tagging

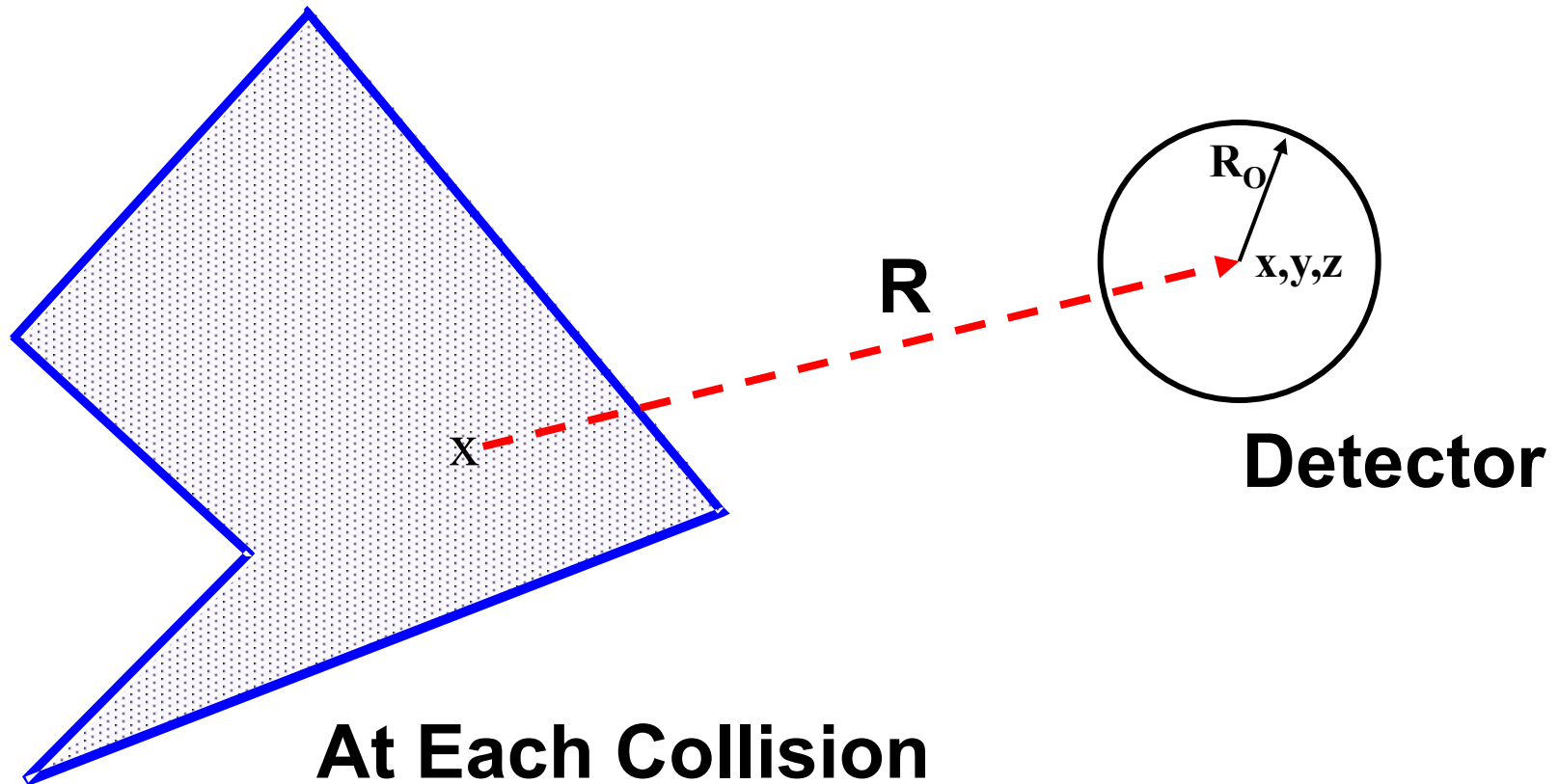
---

**copy c:\MCNP6\EXAMPLES\ata17**

- Add photon type 1 tally to back plane of water block.
- FT TAG option to tally 1.
  - Source
  - Bremstrahlung
  - Fluorescence from both
  - Compton from Oxygen
  - Annihilation photons
  - Everything else (1e10)



# Point Detectors



**At Each Collision**

$$\Phi = Wp(\mu)e^{-\lambda} / (2\pi R^2)$$

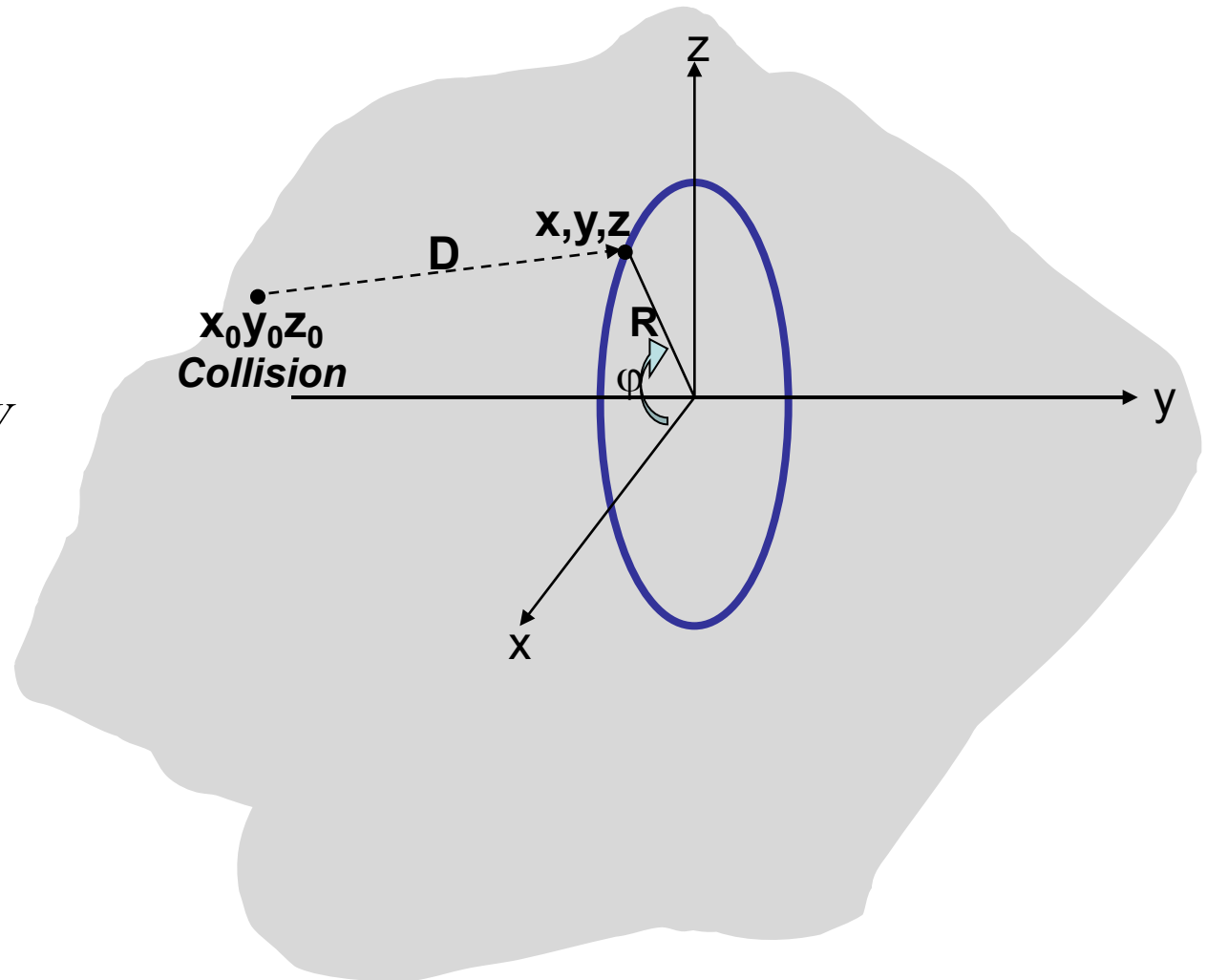
# Ring Detector

Sample from:

$$\xi = \frac{C}{2\pi} \int_{-\pi}^{\varphi} \frac{d\varphi'}{R^2}$$

Adjust weight:

$$W' = \frac{D^2(\varphi)}{A} W$$



# Detector Cards

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## Point Detectors

F5:<pl> X Y Z R<sub>0</sub>

## Ring Detectors

F5x:<pl> X R R<sub>0</sub>

F5y:<pl> Y R R<sub>0</sub>

F5z:<pl> Z R R<sub>0</sub>

## Radiography Tallies

FI5:<pl> X<sub>1</sub> Y<sub>1</sub> Z<sub>1</sub> R<sub>0</sub> X<sub>2</sub> Y<sub>2</sub> Z<sub>2</sub> F<sub>1</sub> F<sub>2</sub> F<sub>3</sub>

# Exercise 8: Point Detectors

---

copy c:\MCNP6\EXAMPLES\atal3

- Add a point detector on axis at  $y=99.9$
- Run the Problem, look at the output

ltally 5 nps = 100000  
 tally type 5 particle flux at a point detector. units 1/cm\*\*2  
 particle(s): photon

detector located at x,y,z = 0.00000E+00 9.99000E+01 0.00000E+00  
 2.27358E-05 0.0121

detector located at x,y,z = 0.00000E+00 9.99000E+01 0.00000E+00  
 uncollided photon flux  
 0.00000E+00 0.00000

detector score diagnostics		cumulative	tally	cumulative
times	average score	fraction of	per	fraction of
		transmissions	history	total tally
	1.00000E-01	30490	6.85395E-07	0.03015
	1.00000E+00	53640	3.56283E-06	0.18685
	2.00000E+00	500	1.37010E-07	0.19288
	5.00000E+00	181	1.52987E-07	0.19961
	1.00000E+01	2116	3.68268E-06	0.36158
	1.00000E+02	3498	1.42777E-05	0.98957
	1.00000E+03	0	0.00000E+00	0.98957
	1.00000E+38	0	0.00000E+00	0.98957
before dd roulette		1384	2.37164E-07	1.00000

average tally per history = 2.27358E-05  
 (largest score)/(average tally) = 5.33468E+01

largest score = 1.21288E-03  
 nps of largest score = 57279

score contributions by cell

	cell	misses	hits	tally per history	weight per hit	
	3	13	55294	91809	2.27358E-05	2.47642E-05
	4	14	100000	0	0.00000E+00	0.00000E+00
	total	155294	91809	2.27358E-05	2.47642E-05	

score misses

russian roulette on pd	0
psc=0.	100564
russian roulette in transmission	54730
underflow in transmission	0
hit a zero-importance cell	0
energy cutoff	0

lanalysis of the results in the tally fluctuation chart bin (tfc) for tally 5 with nps = 100000 print table  
160

normed average tally per history = 2.27358E-05	unnormed average tally per history = 2.27358E-05
estimated tally relative error = 0.0121	estimated variance of the variance = 0.0004
relative error from zero tallies = 0.0044	relative error from nonzero scores = 0.0113
number of nonzero history tallies = 34012	efficiency for the nonzero tallies = 0.3401
history number of largest tally = 46298	largest unnormalized history tally = 1.29479E-03
(largest tally)/(average tally) = 5.69493E+01	(largest tally)/(avg nonzero tally)= 1.93696E+01
(confidence interval shift)/mean = 0.0001	shifted confidence interval center = 2.27384E-05

if the largest history score sampled so far were to occur on the next history, the tfc bin quantities would change as follows:

estimated quantities	value at nps	value at nps+1	value (nps+1)/value (nps)-1.
mean	2.27358E-05	2.27485E-05	0.000559
relative error	1.21135E-02	1.21195E-02	0.000496
variance of the variance	4.39517E-04	4.42102E-04	0.005882
shifted center	2.27384E-05	2.27384E-05	0.000000
figure of merit	2.00086E+05	1.99888E+05	-0.000992

the estimated slope of the 200 largest tallies starting at 7.70242E-04 appears to be decreasing at least exponentially.

the large score tail of the empirical history score probability density function appears to have no unsampled regions.



=====

results of 10 statistical checks for the estimated answer for the tally fluctuation chart (tfc) bin of tally 5

tfc bin behavior	--mean-- behavior	-----relative value	error----- decrease	----- decrease rate	----variance of the variance---- value	decrease	decrease rate	--figure of merit-- value	behavior	-pdf- slope
desired	random	<0.05	yes	1/sqrt(nps)	<0.10	yes	1/nps	constant	random	>3.00
observed	random	0.01	yes	yes	0.00	yes	yes	constant	random	10.00
passed?	yes	yes	yes	yes	yes	yes	yes	yes	yes	yes

=====

this tally meets the statistical criteria used to form confidence intervals: check the tally fluctuation chart to verify.  
the results in other bins associated with this tally may not meet these statistical criteria.

estimated asymmetric confidence interval(1,2,3 sigma): 2.2463E-05 to 2.3014E-05; 2.2187E-05 to 2.3289E-05; 2.1912E-05 to 2.3565E-05  
estimated symmetric confidence interval(1,2,3 sigma): 2.2460E-05 to 2.3011E-05; 2.2185E-05 to 2.3287E-05; 2.1910E-05 to 2.3562E-05

$$fom = (\text{histories/minute}) * (f(x) \text{ signal-to-noise ratio}) ** 2 = (2.936E+06) * (2.611E-01) ** 2 = (2.936E+06) * (6.815E-02) = 2.001E+05$$



# Recommendations

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- **Read output file carefully:**
  - **Understand all warning messages;**
  - **Ensure cross section tables are the ones you wanted;**
  - **Check source with 1<sup>st</sup> 50 histories;**
  - **Check summary to ensure problem is reasonable;**
  - **Check convergence.**
- **Use PRINT card;**
- **Use FC, FQ, TF;**
- **Cross compare with multiple estimators and summary table.**

# Special Tallies

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- Mesh Tallies
- Radiography

# MCNP6 TMESH Tallies

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**There are 4 types of MCNP6 mesh tallies :**

**Type 1:       Track Averaged Mesh Tally**

**Type 2:       Source Mesh Tally**

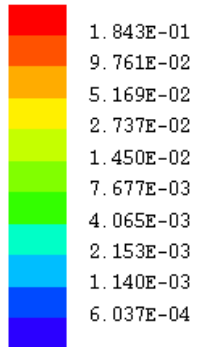
**Type 3:       Energy Deposition Mesh Tally**

**Type 4:       DXTRAN Mesh Tally**

/09/13 15:06:06

0.000000, 1.000000)  
0.000000, 0.000000)

0.00, 20.14)  
40.97, 40.97)

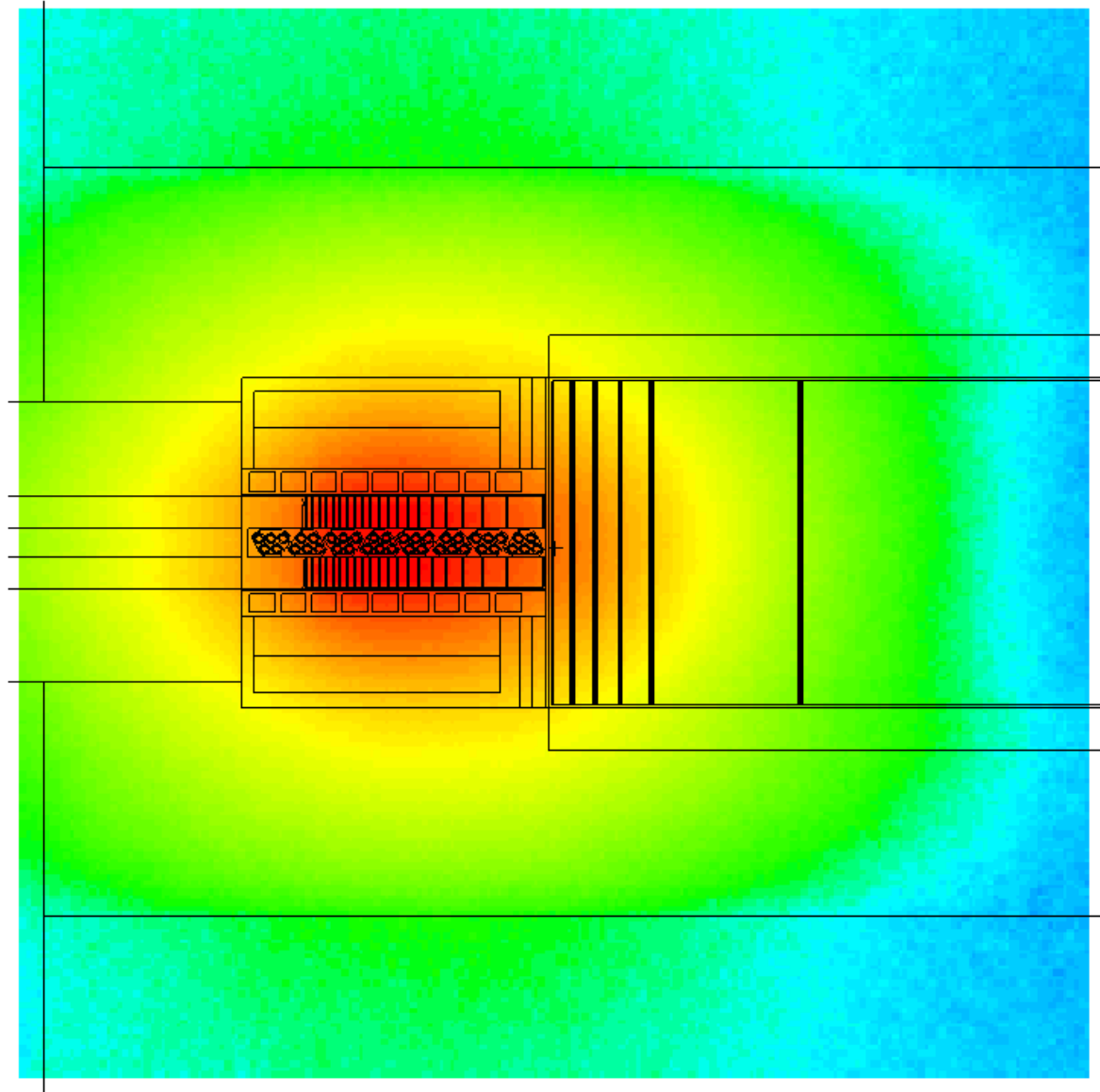


cel 432

cell 432

0.00, 20.14

Restore	CellLine
ROTATE	
SCALES 0	LEVEL
YZ	ZX
L1 off	L2 off
	LEGEND on



# Track Averaged Mesh Tally (type 1)

---

FORM: (R,C,S)MESHn:<pl>      keyword = value  
n = 1, 11, 21, 31,... (note, number must not duplicate  
one used for an 'F1' tally)  
<pl> is a particle type. There is no default.

Example:

```
tmesh
```

```
  rmesh1:n flux
```

```
    cora1 -15.0 99i 15.0
```

```
    corb1 -15.0 15.0
```

```
    corc1 -30.5 99i 30.5
```

```
endmd
```



# Track-Averaged Mesh Tally

---

## Keyword

## Description

<b>TRAKS</b>	Tally the number of tracks through each mesh volume. No values accompany the keyword
<b>FLUX</b>	Tally the average fluence (particle weight times track length divided by volume) in units of number/cm <sup>2</sup> . If the source is considered to be steady state in particles per second, then the value becomes flux in number/cm <sup>2</sup> -s
<b>TRANS</b>	Translate or rotate the mesh according to a specified TR card. This keyword must be followed by a single reference to a TR card.

### Additional keywords:

**DOSE, POPUL, PEDEP, MFACT**

# Source Mesh Tally (type 2)

---

Form: (R,C,S)MESHn <pl<sub>1</sub>> <pl<sub>2</sub>>...<pl<sub>n</sub>> trans = #

n = 2, 12, 22, 32, ...(note, number must not duplicate one used for an 'F2' tally)

<pl> = particle type(s) (Up to 10 allowed)

*Example: Source Mesh tally*

*tmesh*

*RMesh2 n h*

*cora2 -15.0 99i 15.0*

*corb2 -15.0 15.0*

*corc2 -30.5 99i 30.5*

*endmd*

# MCNP6 Mesh Tally Plotting

---

- **From MDATA files**
  - Use gridconv and postprocessor (e.g Moritz, Tecplot, PAW, etc.)

**OR,**

- **From MCTAL files make a contour plot**
  - MCNP6 z
  - rmc = <mctal filename>
  - tal n free ik contour 5 95 10 %
  - **this tells MCNP to plot tally “n”, set the plot indices to your mesh tally coordinates (ik=xz), contour colors where blue=5<95=red, with 10 percent interpolates in between.**

**OR,**

—



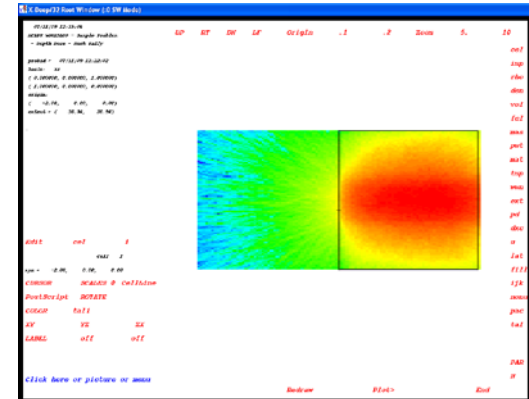
# Mesh Tally Plots

## Superimposed on problem geometry!

- **From INP file during run:**
  - `mplot freq 5000 PLOT ex 40 py 4 la 0 1 tal color on la 0 0`  
(see manual for mplot command detail)

OR,

- **From runtpe:**
  - `mcnp6 z run = <runtpe filename>`
  - `<mcplot> plot $` brings up the geometry plotter
  - `[buttons] tal, N, color`



# Mesh Tally Exercise

---

**Copy C:\MCNP6\EXAMPLES\atal9**

- 1. Plot and understand the geometry.**
- 2. Add a rectangular flux mesh tally for protons and neutrons within the water. Use one bin in the "y" direction.**
- 3. Add a rectangular source mesh tally for protons and neutrons within the water.**
- 4. Plot your results with the MCNP plotter.**

# Plotting the Mesh Tally

## MCNP6 Z

MCNPLOT> **runtpc talmhr**

MCNPLOT> **plot**

Click on **tal**

Click on **color** twice

Click on **ZX**

Turn surf labels off

Zoom in

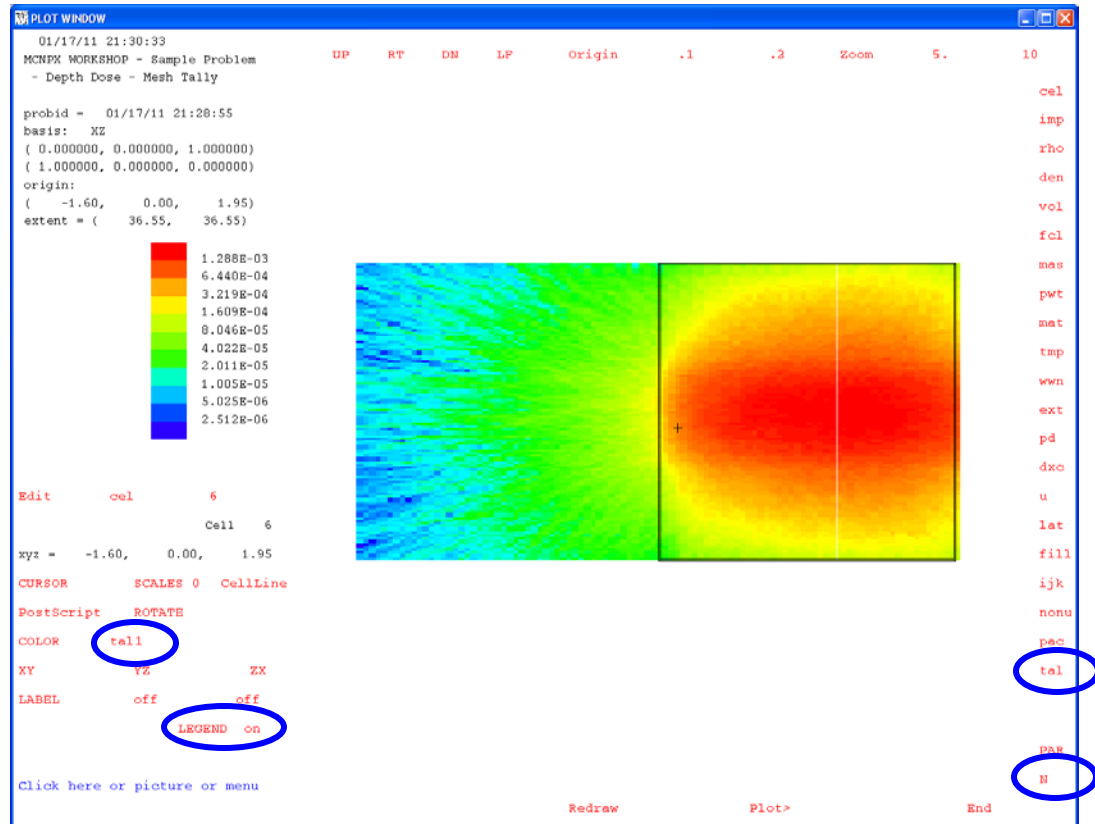
Click on **Legend**

To plot other tallies:

Click on **tal**

Click on **N** to cycle through tallies

Click **redraw**



# Energy Deposition Mesh Tally (type 3)

---

General Form:

**(R,C,S)MESHn keyword**

***n = 3, 13, 23, 33, ...***

***Example: Mesh tally of total energy deposited, all sources***

***tmesh***

***RMesh3 total***

***cora3 -15.0 99i 15.0***

***corb3 -15.0 15.0***

***corc3 -30.5 99i 30.5***

***endmd***

# Some type 3 Mesh Tally keywords

---

## Keyword

## Description

### **TOTAL**

If TOTAL appears on the input line, score energy deposited from any source. (DEFAULT)

### **DE/DX**

If DE/DX appears on the input line, score ionization from charged particles.

### **RECOL**

If RECOL appears on the input line, score energy transferred to recoil nuclei above tabular limits.

**Additional keywords** TLEST, DELCT, MFACT, NTERG, TRANS

(see the manual)

# Mesh Plot Contour Command

---

**FORM: CONTOUR [cmin cmax cstep] [commands]**

All command entries are optional

<b>cmin</b>	minimum contour value
<b>cmax</b>	maximum contour value
<b>cstep</b>	number of contour steps
<b>% or pct</b>	interpret step values as percentages
<b>log</b>	step values logarithmic with cstep interpolates
<b>All</b>	contours normalized to min and max values of entire tally
<b>noall</b>	contours normalized to min and max values of contour slice (FIXED command)
<b>line/noline</b>	do/don't draw lines around contours
<b>color</b>	make color contour plot
<b>nocolor</b>	contour lines only

# Mesh Plot Contour Command

---

**FORM:** **Contour** [cmin cmax cstep] [commands]

## EXAMPLES

### **CONTOUR 5 95 10 & line color**

There will be 10 contour lines at 5%, 15%,...95% of the maximum value.

Lines will be drawn around the colored contours as in Figure 1.

Note: this is the default setting

### **CONTOUR 1e-4 1e-2 12 log**

There will be 12 contour lines logarithmically spaced between 1e-4 and 1e-2

# DXTRAN Mesh Tally (type 4)

---

General Form: **(R,C,S)MESHn:<p/> trans = #**

**n = 4, 14, 24, 34, ...** (note, number must not duplicate one used for an 'F4' tally)

**<p/>** is a particle type. There is no default.

**\*** use \* for DXTRAN; omit \* for F5

**trans** must be followed by a single reference to a TR card that can be used to translate and/or rotate the entire mesh. Only one TR card is permitted with a mesh card.



# MCNP6 FMESH Tally

---

```
FMESH4:n GEOM=cyl ORIGIN= -100 0 0
      IMESH=5 10                      IINTS=5 2
      JMESH= 100 200                  JINTS 10 5
      KMESH .5 1                      KINTS=1 2
      AXS= 1 0 0 VEC=0 1 0          OUT=ij
```

Out = cf, ij, jk, ik ; GEOM = rec, cyl, xyz, rzt

- *MCNP6 has many more options and GEOM = sph, rpt*
- *MCNP6 allows E, T, FM, etc.*

# MCNP6 FMESH Tally

---

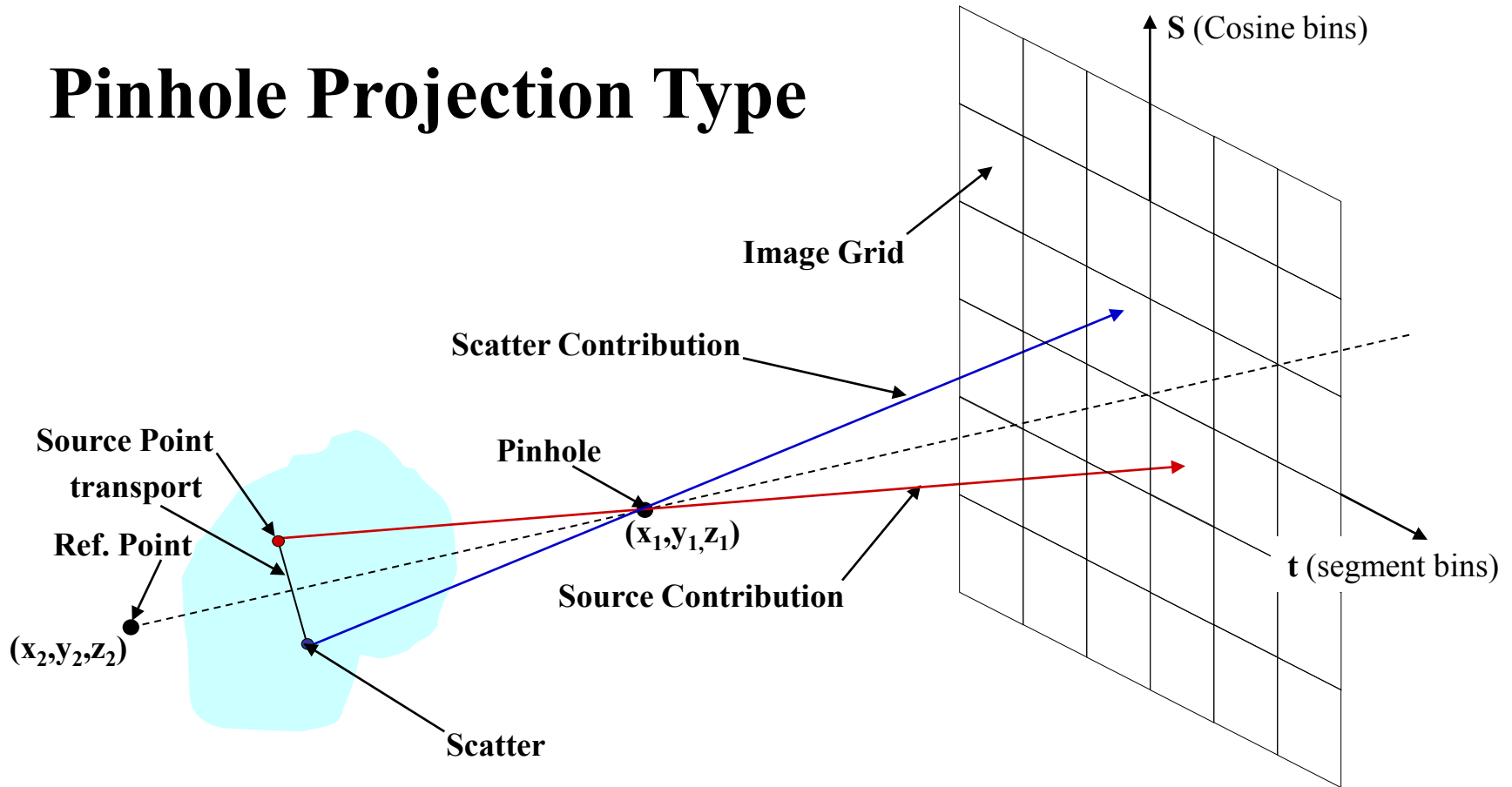
```
fmesh504:n geom=rec origin -400 -400 0  
  imesh 400 iints 99  
  jmesh 400 jints 99  
  kmesh 400 kints 1
```

```
mplot freq 5000 fmesh 504
```

***In geometry plot: click fmesh***

# The Radiography Tally

## Pinhole Projection Type



# RADIOGRAPHY TALLY

## (Pinhole Projection type)

---

General Form:

**FIPn:** *<p/>*  $X_1 Y_1 Z_1 R_0 X_2 Y_2 Z_2 F_1 F_2 F_3$

**FSn** -20. 99i 20.     \$ establishes an image grid with

**Cn** -20. 99i 20.     \$ 100 Seg. x 100 Cos. bins

**n** is the tally number and must end with a 5 since this is a detector-type tally.

*<p/>* is the particle type for the tally.     **Neutrons or photons only!**

(see next slide for explanation of Argument elements)

# Pinhole Radiography Arguments

---

$X_1, Y_1, Z_1$  The coordinates of the pinhole.

$R_0$  Pinhole Radius.

Note, neither the pinhole nor the grid should be located within a highly scattering media.

$X_2, Y_2, Z_2$  The reference coordinates that establish the reference direction cosines for the normal to the detector grid. This direction is defined as being from  $X_2, Y_2, Z_2$  to the pinhole at  $X_1, Y_1, Z_1$ .

$F_1$  If  $F_1 > 0$ , the radius of a cylindrical collimator, centered on and parallel to the reference direction, which establishes a radial field of view through the object.

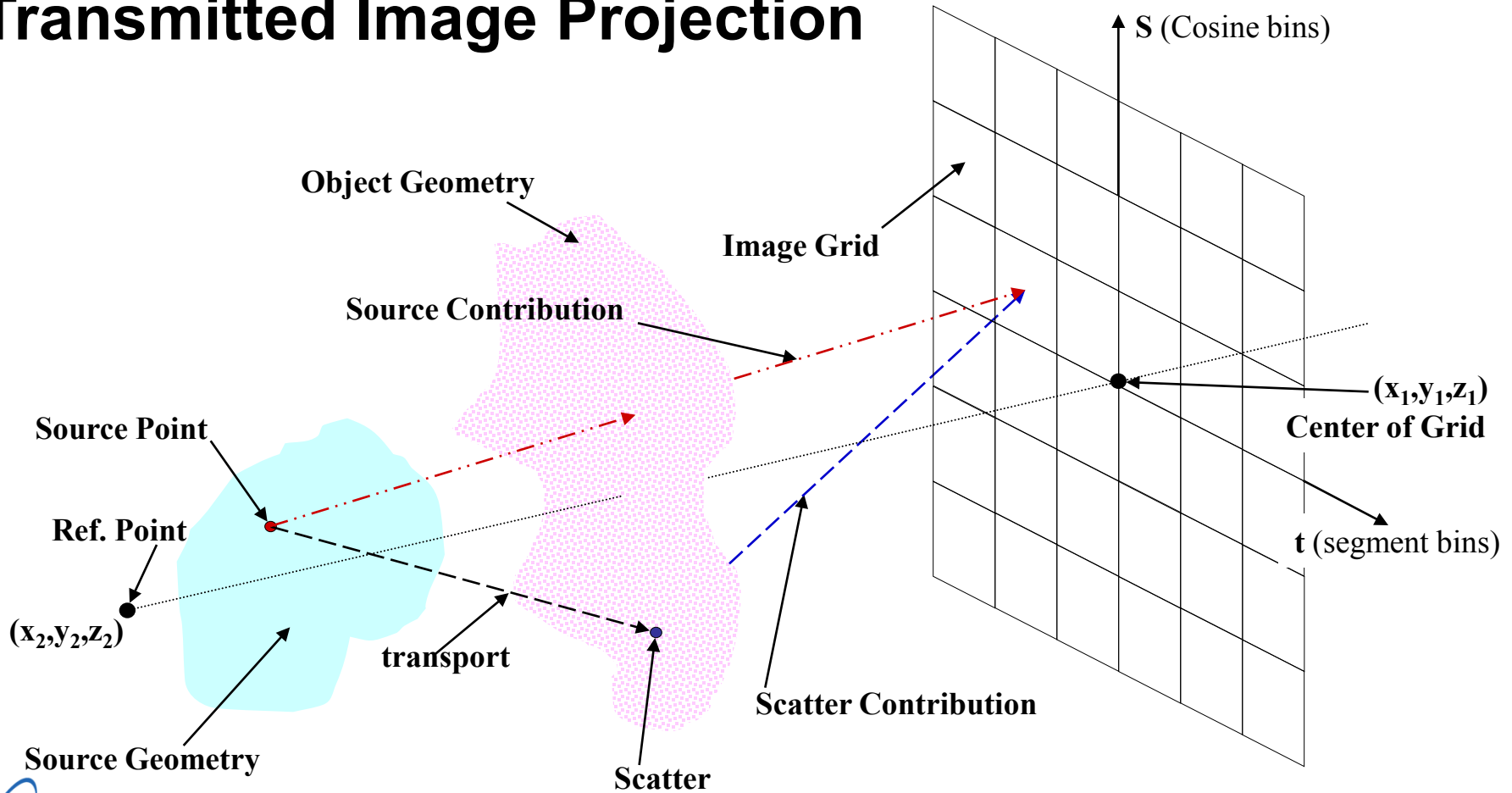
$F_2$  The radius of the pinhole perpendicular to the reference direction.

- $F_2 = 0$  represents a perfect pinhole
- $F_2 > 0$  the point through which the particle contribution will pass is picked randomly. This simulates a less-than-perfect pinhole.

$F_3$  The distance from the pinhole at  $X_1, Y_1, Z_1$  to the detector grid along the direction established from  $X_2, Y_2, Z_2$  to  $X_1, Y_1, Z_1$ , and perpendicular to this reference vector.

# Radiography Tally

## Transmitted Image Projection





# Radiography Tally

## Transmitted Image Projection Type

General Form: **FI(R/C)n:<pl> X<sub>1</sub> Y<sub>1</sub> Z<sub>1</sub> R<sub>0</sub> X<sub>2</sub> Y<sub>2</sub> Z<sub>2</sub> F<sub>1</sub> F<sub>2</sub> F<sub>3</sub>**

**FI**R is used to establish a grid on a plane surface

**FI**C is used to establish a grid on a cylindrical surface.

- n** = the tally number and must end with a 5 since this is a detector type tally.
- <pl>** = the particle type for the tally. (**N or P only**)
- X<sub>1</sub> Y<sub>1</sub> Z<sub>1</sub>** = Center of rect. or cyl. grid defined with FS<sub>n</sub> and C<sub>n</sub>
- R<sub>0</sub>** = 0.00
- X<sub>2</sub> Y<sub>2</sub> Z<sub>2</sub>** = reference point defining rectangular grid outward normal or of cylindrical grid axis. May be thought of as the eye of the observer.
- F<sub>1</sub>** = -1/0 Scattered contribution only/Source + scattered contributions
- F<sub>2</sub>** = radial field of view. Cylinder along the axis.
- F<sub>3</sub>** = 0 /1 Contributions to grid bin centers/random positions



# Transmitted Image Projection

NPSMG on the NPS card

---

NPS      NPP NPSMG

NPP      = number of histories requested

NPSMG = number of direct source contributions requested

Example: NPS 100000 60000



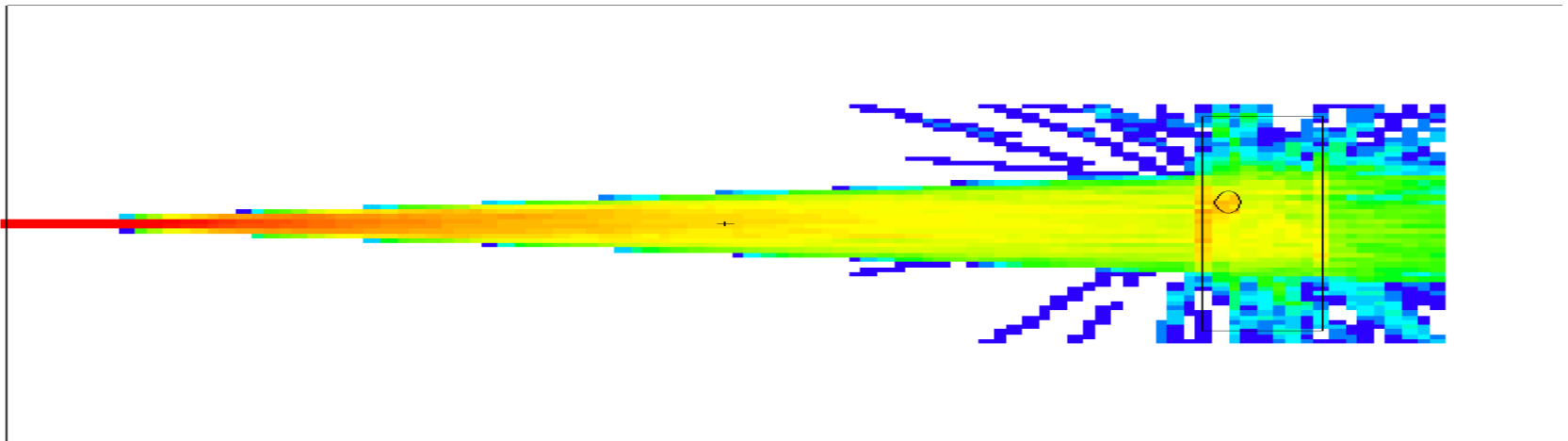
# Exercise Rad 10

Transmitted Image Projection:  $^{235}\text{U}$  sphere in water

---

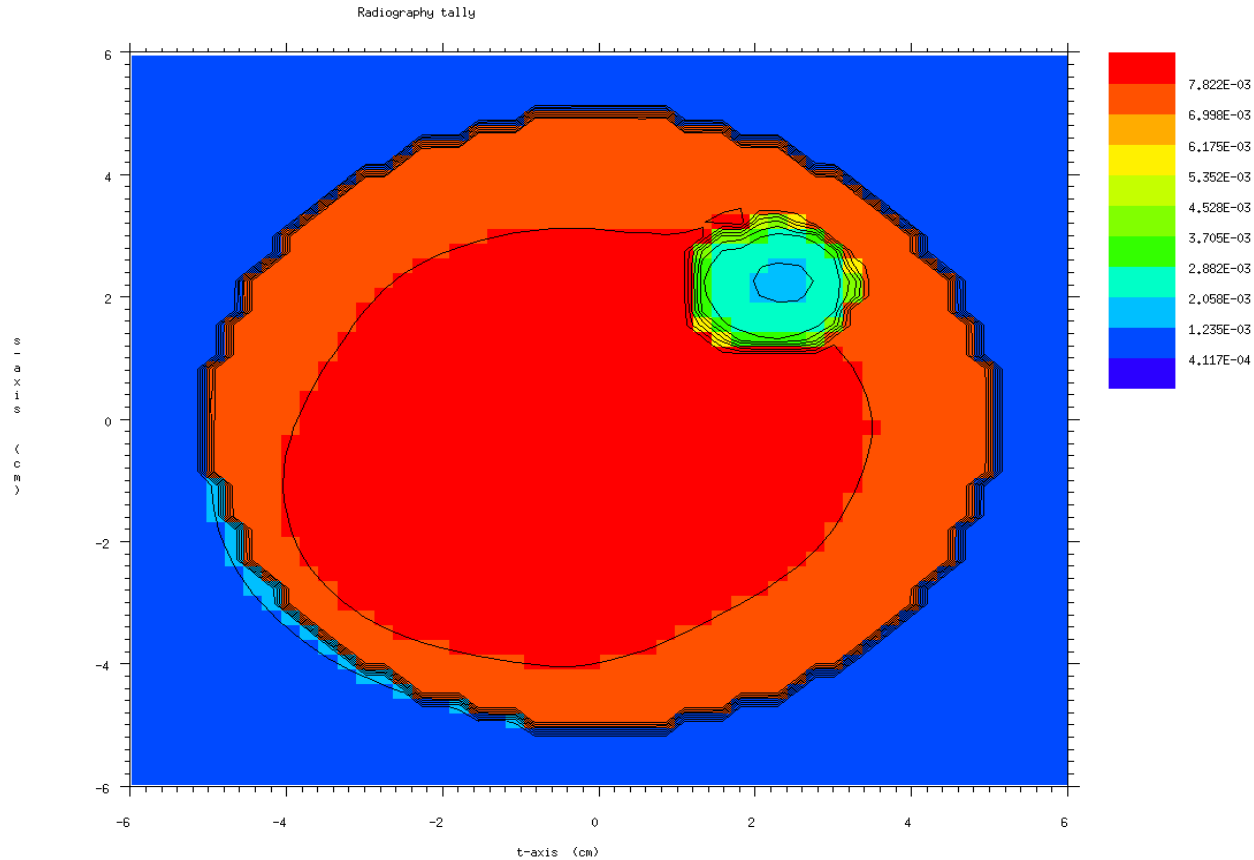
Copy %inputs%\tally\rad10

- 10-cm radius, 10-cm tall water tank
- 2-cm radius  $^{235}\text{U}$  off-center sphere
- 1-MeV photon source 100-cm away  $.999 < \mu < 1.0$  cone
- Radiography tally behind tank



# Exercise Rad 10

Transmitted Image Projection:  $^{235}\text{U}$  sphere in water

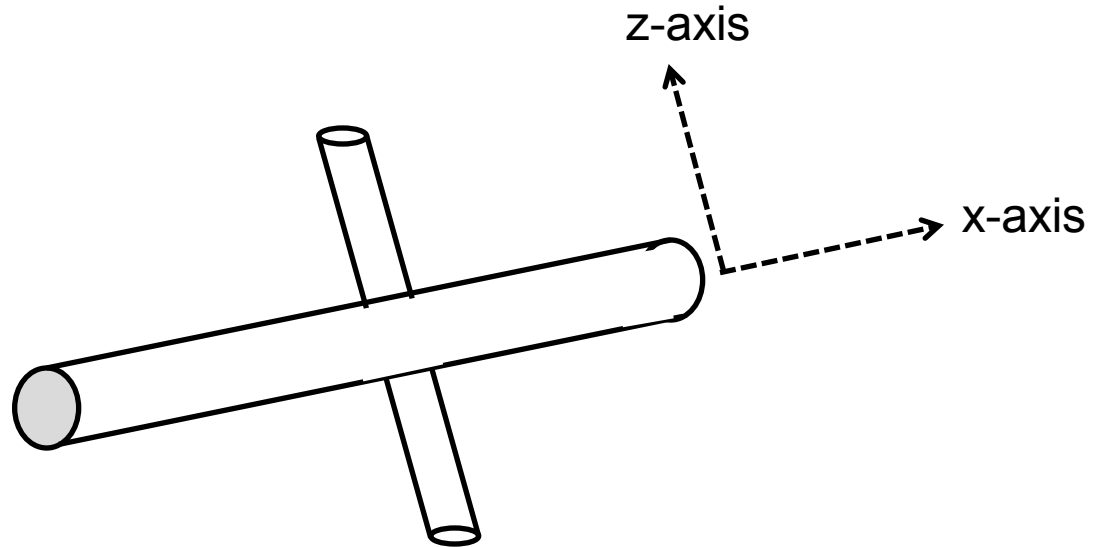


# Exercise Rad 11

## Safeguards Radiography

Larger pipe contains 50% (atom)  $\text{UO}_2$  and 50%  $\text{H}_2\text{O}$ .  $\text{UO}_2$  is 10% (atom) enriched. Use a gram density of 10.0. Inner pipe radius is 1.0 cm and overall length is 40 cm. Place origin at the center of this pipe. Pipe is made of  $^{208}\text{Pb}$  with a thickness of 1.0 mm. Use a gram density of 11.4.

Enclose this geometry in a large sphere. This requires 5 cells and 4 surfaces.



Void pipe through center of larger pipe ( $R=0.5$  cm). Length large enough to pass through larger pipe.

# Exercise Rad 2

## Safeguards Radiography

---

### Copy %inputs%\tally\rad11

- Specify a spontaneous photon (sp) source spread uniformly throughout the HEU solution.
- Run MODE “p” only and turn on delayed gammas (PHYS:P 6th entry).
- Use a cylindrical TI tally around the larger pipe, with the image grid centered at the origin (use 0.001,0,0 due to a bug). Use a cylindrical radius of 10 cm for the image surface. Use 20 segments along the axis of the pipe and 18 angular segments around the outside surface of the pipe (i.e., every 20 degrees).
- Run 500,000 histories. You may encounter a “bad trouble” – think about this for awhile (hint: look at the volumes calculated for each cell).
- Modify the input to increase 235U component to 20% (atom). Generate contour plots of both cases that clearly show the effect of the higher enrichment.
- Can you see the void cross pipe? Why or why not?
- Note the TIC s-axis is always along the cylinder axis. What t-axis was chosen (i.e., what corresponds to  $q=0$ )? Does the cross pipe image help with this?

# Exercise Rad 2

## Safeguards Radiography

