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Recent Nuclear Criticality Safety Program Technical Accomplishments ANS Winter Meeting 2011 - Washington, DC LA-UR-11-06130

# **MCNP Developments**

Recent Monte Carlo Progress in Support of the US DOE Nuclear Criticality Safety Program

# Forrest Brown & Brian Kiedrowski



Monte Carlo Codes, XCP-3

Los Alamos National Laboratory





### **MCNP Developments**

Forrest Brown & Brian Kiedrowski, XCP-3, LANL

This presentation covers recent progress in development and support of the MCNP Monte Carlo code during FY 2011 in support of the US DOE/NNSA Nuclear Criticality Safety Program.

Activities and accomplishments are summarized in six major areas:

- MCNP5-1.60 Release
- MCNP6 Status
- Verification / Validation
- User Support & Training
- R&D Work in Progress
- Future Release Plans



US DOE/NNSA Nuclear Criticality Safety Program –

What have we done for you lately ?

- MCNP5-1.60 Release
- MCNP6 Status
- Verification / Validation
- User Support & Training
- R&D Work in Progress
- Future Release Plans



## Chronology

- Development:
- Extensive testing & V&V: Summer 2010
- Sent to RSICC:
- RSICC release:

Fall 2009 – Spring 2010

- September 2010
- October 2010, July 2011

## Focus

- Stability + reliability for criticality calculations
- Support for latest computers multicore, Windows/Mac/Linux, 32/64 bit
- Rigorous, extensive code V&V
- A few new features, many minor bug-fixes
- Last version of MCNP5 want strong base for going forward with MCNP6
- Notable
  - Most rigorous & extensive MCNP testing ever
  - Over 5,000 hr computer time for V&V, mostly on criticality problems
  - First release of adjoint-weighted tallies (kinetics parameters)



- Adjoint-weighted Tallies for Point Kinetics Parameters
  - $-\beta_{eff}, \Lambda_{eff}, Rossi-\alpha$  using continuous-energy Monte Carlo
  - Adjoint-weighted tallies from iterated fission probability
- Mesh Tallies for Isotopic Reaction Rates
  - Previously, could only do flux, dose, material reaction rates
  - Important extension to specific isotopes
- Increased Limits for Geometry, Tally, and Source Specifications
  - Allow up to 100M for cell, surface, material specs. Previous 100K limit
  - Complex cell spec up to 9999 items, previous 999 limit
- General
  - Improved threading on multicore computers, Mac/Linux/Windows
  - 12 other minor enhancements to code
  - 30 minor bug-fixes (none affect results for criticality calculations)

Testing + V&V Suites

REGRESSION VALIDATION\_CRITICALITY VERIFICATION\_KEFF VALIDATION\_SHIELDING KOBAYASHI [new] POINT KINETICS [new] - 66 installation & regression test problems

MG

- VALIDATION\_CRITICALITY 31 ICSBEP Handbook cases
  - 75 analytic problems, exact results
  - 19 shielding/dose problems vs experiment
  - void & duct streaming, point detectors
  - adjoint weighted Rossi- $\alpha$ ,  $\beta_{eff}$ ,  $\Lambda_{eff}$

- Computers
  - Mac / Linux / Windows, 32 / 64 bit
  - Sequential, threads, MPI, threads+MPI
  - Over 5,000 hr computer time
  - 2 students full-time + 2.5 staff part-time for 3 months
- Criticality calculations
  - Tested with ENDF/B-VI & ENDF/B-VII.0
  - All results should match previous versions of MCNP5

/lonte Carlo Code

#### Monte Carlo Codes XCP-3, LANL

### MCNP Criticality Validation Suite, Results on Mac OS X for ENDF/B-VII.0

	Experiment		MCNP5-1.51		MCNP5-1.60							
U233 Benchmarks												
JEZ233	1.0000	(10)	0.9989	(6)	0.9989	(6)						
FLAT23	1.0000	(14)	0.9990	(7)	0.9990	(7)						
UMF5C2	1.0000	(30)	0.9931	(6)	0.9931	(6)						
FLSTF1	1.0000	(83)	0.9830	(11)	0.9830	(11)						
SB25	1.0000	(24)	1.0053	(10)	1.0053	(10)						
ORNL11	1.0006	(29)	1.0018	(4)	1.0018	(4)						
HEU Benchmarks												
GODIVA	1.0000	(10)	0.9995	(6)	0.9995	(6)						
<b>TT2C11</b>	1.0000	(38)	1.0018	(8)	1.0018	(8)						
FLAT25	1.0000	(30)	1.0034	(7)	1.0034	(7)						
GODIVR	0.9985	(11)	0.9990	(7)	0.9990	(7)						
UH3C6	1.0000	(47)	0.9950	(8)	0.9950	(8)						
ZEUS2	0.9997	(8)	0.9974	(7)	0.9974	(7)						
SB5RN3	1.0015	(28)	0.9985	(13)	0.9985	(13)						
ORNL10	1.0015	(26)	0.9993	(4)	0.9993	(4)						

	Experiment		MCNP5-1.51		MCNP5-1.60							
IEU Benchmarks												
IMF03	1.0000	(17)	1.0029	(6)	1.0029	(6)						
BIGTEN	0.9948	(13)	0.9945	(5)	0.9945	(5)						
IMF04	1.0000	(30)	1.0067	(6)	1.0067	(6)						
ZEBR8H	1.0300	(25)	1.0195	(6)	1.0195	(6)						
ICT2C3	1.0017	(44)	1.0037	(7)	1.0037	(7)						
STACY36	0.9988	(13)	0.9994	(6)	0.9994	(6)						
LEU Benchmarks												
BAWXI2	1.0007	(12)	1.0013	(7)	1.0013	(7)						
LST2C2	1.0024	(37)	0.9940	(6)	0.9940	(6)						
Pu Bench	marks											
JEZPU	1.0000	(20)	1.0002	(6)	1.0002	(6)						
JEZ240	1.0000	(20)	1.0002	(6)	1.0002	(6)						
PUBTNS	1.0000	(30)	0.9996	(6)	0.9996	(6)						
FLATPU	1.0000	(30)	1.0005	(7)	1.0005	(7)						
THOR	1.0000	(6)	0.9980	(7)	0.9980	(7)						
PUSH2O	1.0000	(10)	1.0012	(7)	1.0012	(7)						
HISHPG	1.0000	(110)	1 0122	(5)	1.0122	(5)						
PNL2	1.0000	(65)	1.0046	(9)	1.0046	(9)						
PNL33	1.0024	(21)	1.0065	(7)	1.0065	(7)						

## MCNP5-1.60 Release

(5)





## Parallel performance

 MCNP5 has always used hierarchical parallelism with MPI + threading

#### Multicore threading speedups:





#### Hardware

- Lobo 16 threads/node
  - 4 x Quad-core AMD Opteron, 2.2 GHz, 32 GB memory
- Mac Pro 8 threads

2 x Quad-core Intel Xeon, 3GHz, 8 GB memory

#### Software

MCNP5, Intel-10 F90, "-O1 -openmp"

#### **MCNP Criticality Calculation**

BAWXI2 benchmark, kcode 25000 1 10 204

MCNP6 Status (1)



- MCNP6 beta release sent to RSICC for a limited set of beta testers
- MCNP6 full release by RSICC expected in 2012
- Culminates 5 years of effort combining features of MCNPX into MCNP5
- MCNP5 & MCNPX are now frozen future development will occur in MCNP6





Monte Carlo Codes

XCP-3. LANI



## MCNP6 contains:

- MCNP6 = development version of MCNP at LANL, since 2004
- Includes:
  - All MCNP5-1.60 capabilities (mpi + threads)
  - All MCNPX 2.7.0 capabilities (mpi)
  - Adjoint-weighted perturbation estimators
  - CINDER 2008 decay & depletion
  - High energy protons & magnetic fields, for proton radiography
  - Unstructured mesh, for linking with ABAQUS
  - Structured mesh, for linking with PARTISN
- MCNP6 in (very) limited beta release
  - Recipients are active collaborators and sponsors
  - Full beta access within LANL and LLNL

Monte Carlo Codes XCP-3, LANL

## MCNP6 – all particles & all energies, using best data + models + theory



### • Recent physics improvements:

- Photon induced fission multiplicity
- Characteristic muonic X-rays
- Exact delayed gamma emissions
- Visible light
- Improved photoatomic form factors
- Upgrades to CEM & LAQGSM 3.03
- GEF photofission yield

#### Incorporates other codes:

- CINDER burnup & decay LANL
- ITS

HETC

- electron transport
- LAHET high energy transport LANL
- CEM high energy transport LANL
- LAQGSM high energy transport LANL
- MARS high energy transport FNAL
  - high energy transport ORNL

SNL



# MCNP & MCNPX teams have adopted MCNP6 as the base for all future development

- To go from Beta release to Production release:
  - Assurance of reliability and accuracy for criticality
  - Assurance of reliability and accuracy for other apps
  - Comparable performance
  - Complete documentation
- Future Work
  - Cleanup coding style
  - Remove duplicate features
  - Extend parallel threading capability to new features
  - New Features
- General release through RSICC
  - 2012



- MCNP V&V Suites -- Focus
  - Physics-based V&V
  - Compare to experiment or exact analytic results
  - Part of MCNP permanent code repository & RSICC distribution
  - Automated easy to run & collect results, compare to experiments
- Continuous Test System for MCNP6
  - 3 platforms Linux 32, Linux 64, Windows 64
  - 5 compilers Intel 10+11, PGI 7, Pathscale 3, gfortran
  - Serial, mpi, omp, mpi+omp
  - Array bounds checking
  - 875 problem input files
  - Total: 10,000 runs each night

## Testing + V&V Suites

REGRESSION VALIDATION\_CRITICALITY VERIFICATION\_KEFF VALIDATION\_SHIELDING KOBAYASHI POINT\_KINETICS - 66 installation & regression test problems

**M**C

- **VALIDATION\_CRITICALITY** 31 ICSBEP Handbook cases
  - 75 analytic problems, exact results
  - 19 shielding/dose problems vs experiment
  - void & duct streaming, point detectors
  - adjoint weighted Rossi- $\alpha$ ,  $\beta_{eff}$ ,  $\Lambda_{eff}$
- Recently added V&V Suites

VALIDATION\_CRIT\_EXPANDED -119 ICSBEP Handbook experiments
Rossi Alpha Val. Suite - 12 ICSBEP Handbook experiments
VALIDATION\_LANL\_SB-CS -194 ICSBEP Handbook experiments
VALIDATION\_SHIELDING - improved models & experiment results
High-E proton, heavy ion
Delayed n & γ spectra
Subcritical multiplication
Perturbation verification
Production / depletion

Monte Carlo Codes



 MCNP5 & MCNP6 continue to be used extensively in the development & testing of ENDF/B-VII.0 and ENDF/B-VII.1 nuclear data libraries:

"As noted there were a number of additional "first-ever" features of the ENDF/ B-VII.0 validation effort. There was almost total reliance on continuous-energy Monte Carlo methods. Because these methods can handle even the most complex geometry features of the benchmark models and provide "rigorous" treatment of the neutron physics, the calculational errors are reduced essentially to the statistical errors of the computation and the errors due to nuclear data. And it is now practical to run sufficient neutron histories in the computation to make the former errors negligibly small relative to the latter. Furthermore, the validation calculations relied almost totally on a single code system (NJOY/MCNP). This dominance of a single code system is not viewed as limiting the validation effort given (i) the general acceptance and universal recognition of the high quality of this physics tool and (ii) the availability of sufficient, both historical and current, independent verification of the ENDF/B-VII.0 validation results by other independent code systems, primarily with results of the U.S. Naval Reactor Labs code RACER [281, 282] and RCP01 [283] and the Argonne National Laboratory code VIM [284]."

M.B. Chadwick, et al., *ENDF/B-VII.0: Next Generation Evaluated Nuclear Data Library for Nuclear Science and Technology*, Nuclear Data Sheets, Vol. 107, No. 12, pp 2931-3060 (2006)



- 11,586 copies of MCNP distributed by RSICC, Jan 2001 Oct 2011
- Classes
  - Theory & Practice of Criticality Calculations with MCNP5

FY 2011: Hanford/PNNL, LANL, Y-12FY 2012: INL, PNNL/Hanford, ?Previous: Y-12, PNNL, INL, ANL, LLNL, SRL, LANL

- Introduction to MCNP5 4 classes/year at LANL
- Advanced Variance Reduction 1-2 classes/year at LANL

## Conference Papers & Presentations

- PHYSOR 2010: Monte Carlo workshop - SNA+MC-2010: plenary talk + 3 papers (crit, perturb, parallel) – ANS Winter 2010: (V&V for perturb) 1 paper - M&C 2011: (statistical tests) 1 paper - ANS Summer 2011: 3 papers (stats, V&V, crit suite) - ICNC 2011: (V&V, status, methods) 6 papers – NS&E journal: paper on adjoint-weighted tallies - PNST Journal: 4 papers 17



## MCNP Forum

- User-group beginners & experts, ~1,000 members
- Feedback, bug reports, guidance

## Reference collection

- DVD with 1000+ MB of references on Monte Carlo & MCNP, ~600 items
- Web browser based
- All MCNP6, MCNP5, & previous MCNP code documentation
- Criticality, V&V, adjoints, electrons, detectors, parallel, benchmarks, .....
- Includes 8 half-day Monte Carlo workshops

## University collaborations

- Michigan, New Mexico, Wisconsin, Oregon State, MIT, RPI
- Summer students at LANL
- Participated in ANS 10.7 Standards committee



## **Perturbation Theory**

- MCNP computes perturbations to k using two methods:
  - Differential operator/Taylor series
  - Adjoint-based (new 2010, MCNP6)
    - Allows applications such as additions of impurities, crosssection library comparisons, & material substitutions that were not possible with differential operator

## • V&V efforts show:

- Good agreement when  $\Delta k$ 's in same direction
- When both are apply, methods are complementary (neither is always better)
- Difficulties with scattering cross section perturbations observed

## Continuous-Energy Sensitivity Coefficients

- MCNP6 can produce sensitivity coefficients in continuous-energy
  - Uses adjoint-weighted perturbation techniques
  - Verified against multigroup results
  - Good performance for capture and fission

## Scattering needs improvements

- In theory need double-differential scattering cross sections
- Approximate extensions with scattering laws should be straightforward
- Exact is theoretically possible, but costly

# Boundary Sensitivity Coefficients

- Boundary interface locations (e.g., radius of a sphere) can be uncertain
- Favorite showed theory of computing sensitivity coefficients to interface locations
- Should be extendable to continuous-energy Monte Carlo
  - Necessarily uses "exact" method for scattering sensitivities
  - Work in progress, results to be presented

# Reliability of Large Collections of Tallies

- Criticality alarm design & accident assessment require global flux or dose fields
  - High resolution, >100K mesh elements
  - Can results be trusted?
- Some MCNP statistical checks extended
  - Observe convergence of moments of relative variances
  - Minimal memory cost or compute time
  - Sensitive to large scores in a few mesh
    - Suggests undersampling of problem
- Applications in criticality safety, reactor physics, shielding, etc.

## **Statistical Coverage**

- Implicit assumption in criticality calculations that all regions of problem are adequately covered
  - Easy to satisfy for typical reactors
  - May be difficult for certain criticality safety problems
    - Multiple regions, loosely coupled
    - Some regions more/less reactive
    - Asymmetric coupling between regions



- Revised k<sub>eff</sub> of the World problem
  - 9x9x9 array of Pu spheres in water
  - Central sphere larger, cadmium coated

MGU

Nonte Carlo Codes

XCP-3, I AN

- Loose, asymmetric coupling
- Results of 100 independent trials:
  - Starting guess uniform in all spheres
  - keff and  $\rm H_{src}\,$  always converged
  - 10K particles per cycle:
    - 60% chance of getting wrong k



#### Monte Carlo Codes XCP-3, LANL

# Continuous S(α, β) Scattering For Thermal Neutrons

## S( $\alpha$ , β) thermal neutron scattering

- Accounts for temperature, chemical-, & molecular-binding on collision physics
- Traditional NJOY-MCNP uses discrete energy-angle data, gives ray effects for problems with only a few scatters

## • Continuous $S(\alpha, \beta)$ treatment

- Developed by MacFarlane in early 2000s
- Implemented in MCNP5-1.51 & 1.60
- Never adequately verified/validated

## Recent V&V effort

- A. Pavlou (U.Mich), 2011
- Thorough V&V with ICSBEP benchmarks
- Conclusion: valid for use in crit-safety
- Continuous S(α, β) data to be included with MCNP ENDF/B-VII.1 data libraries

## Free-gas Resonance Scattering For Epithermal Neutrons

## Free-gas scattering model

- For neutrons with energies of a few eV to a few 100 eV, used to account for target nuclide thermal motion
- Traditional: assume constant  $\sigma_{scatter}$
- Resonance scatter can be important for free-gas model







 MCNP mods to include resonance scattering in free-gas model

## **On-The-Fly Doppler Broadening**

- US DOE NEUP project with Univ. Michigan, ANL, LANL (2011-2012)
  - W.R. Martin & students (U.Mich)
  - G. Yesilyurt (ANL)
  - F.B. Brown (LANL)

## Provide general temperature treatment for MCNP

- OTF Doppler broadening
- OTF interpolation for S(a,b) thermal
- OTF interpolation for unresolved resonance probability tables
- Provides continuous termperature capability, without precomputing thousands of cross-section datasets

## CMFD & Fission Matrix Convergence Acceleration

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## Hybrid methods to improve convergence of fission source distribution for criticality

- Use low-order deterministic method to accelerate global convergence of MC source distribution
- Parameters for low-order solution obtained from accurate MC

## CMFD / MC

•

- Coarse-Mesh Finite Difference method used for low-order
- Low-order solutions used to bias source sampling in MCNP

## Fission matrix

- Computed directly during MC
- Fission matrix eigenvector used to bias source sampling in MCNP

Monte Carlo Codes

#### Monte Carlo Codes XCP-3, LANL

# Improvements to Parallel MPI & Threading

- For criticality calculations
  - Reduce the amount of data exchanged at MPI rendezvous
- MPI improvements
  - Automatic chunking of large transfers
  - Convert from buffered MPI message arrays to in-place
  - Asynchronous MPI messages
  - Improve Fortran/C interface

## OpenMP threading improvements

- Replace private thread-safe storage for certain large arrays by OpenMP critical section
- Use OpenMP atomic operations with shared tally arrays

## Parallel MC for Exascale Computers

- Exascale computers are coming
  - Millions of cpu cores
  - Reduced memory/cpu-core

## Need new parallel approach

## Parallel on particles + Distributed data

- Particles distributed among nodes
- Fetch data remotely from other nodes as needed by particles (do not move particles to data)
- Eliminate synchronization & particle motion among compute nodes



- MCNP6 = MCNP5 + MCNPX merger •
- Impact on Criticality Calculations  $\rightarrow$  none
  - All KCODE criticality features same as for MCNP5
  - Matches results with MCNP5 for criticality suites
- Monte Carlo team will support MCNP6, no new features or releases of MCNP5 or MCNPX
- MCNP6 is coming
  - **1Q CY 2011 very limited distribution** – Beta-0 release:
  - Beta-1 release:
  - Production release:

- 4Q CY 2011 general beta testing
  - CY 2012 RSICC release

# We need to plan for MCNP5 $\rightarrow$ MCNP6 transition over the next few years



# **Questions**?