

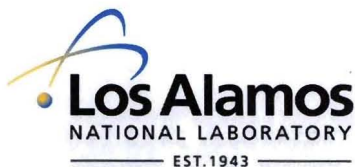
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Title: MCNP6 Hybrid Geometry: Overview (U)

Author(s): Roger L. Martz

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# MCNP6 Hybrid Geometry

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## Overview

# ABSTRACT

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**This presentation provides an overview of the new hybrid geometry capability in MCNP6. The modeling paradigm, some implementation, and user requirements are discussed. The new MCNP input cards are presented. A simple example is shown to illustrate some of the relevant ABAQUS® features. Recent accomplishments and near term goals are mentioned.**

## Outline

- Overview of the capability
- A quick view of the new MCNP Input Cards
- Concrete cylinder example / sample results from ABAQUS
- Recent accomplishments
- Near term goals

# Capability Overview

## Objectives of these next slides:

- Provide the user with a basic understanding of
  - the modeling paradigm used in this hybrid approach
  - some implementation & user requirements

## Embedded mesh

- **Mesh geometry co-exists with MCNP cell-based geometry.**
  - read from separate input file
- **Implemented as a universe.**
- **Ultimately, “many” instances of the same mesh or instances of “several” different mesh.**

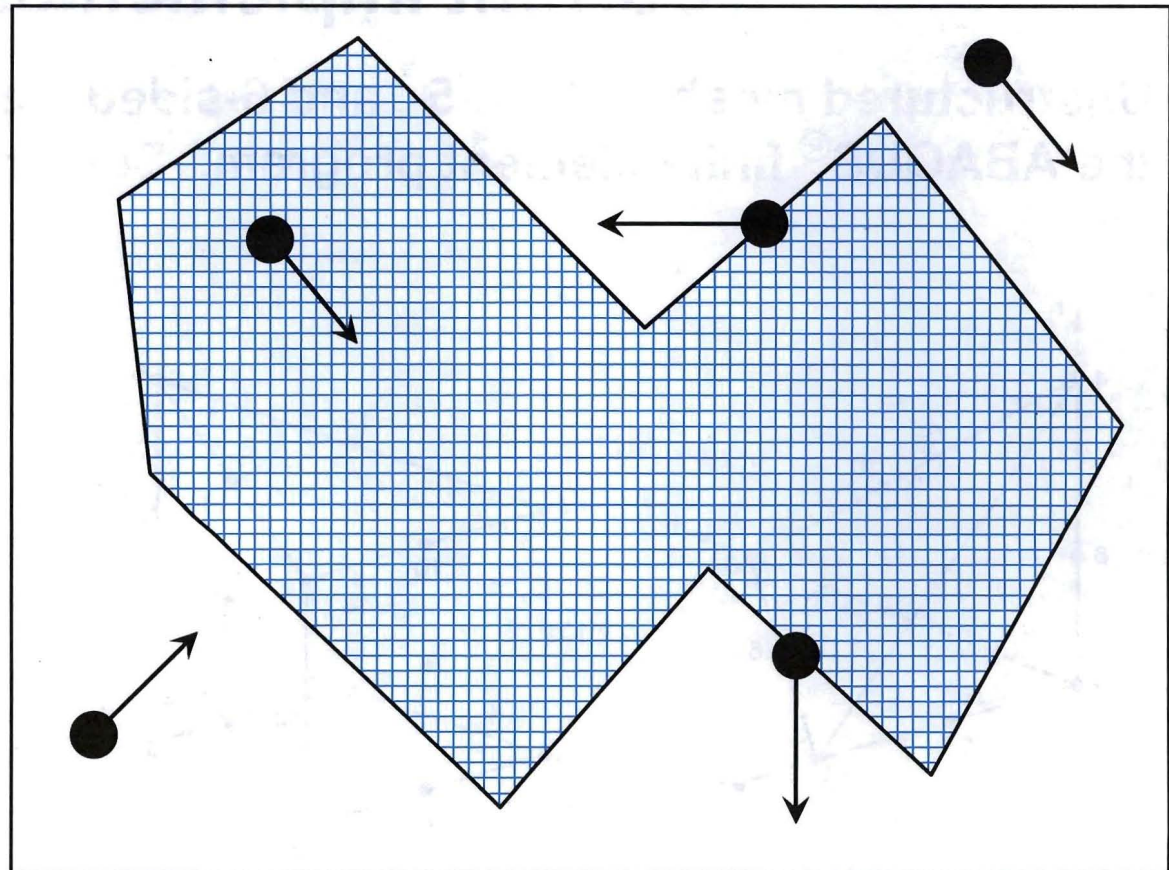
# Some Geometry Requirements

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## Irregular mesh body

Tracking considerations:

1. Outside hitting
2. Outside missing
3. Inside
4. Leaving cleanly
5. Re-entrant

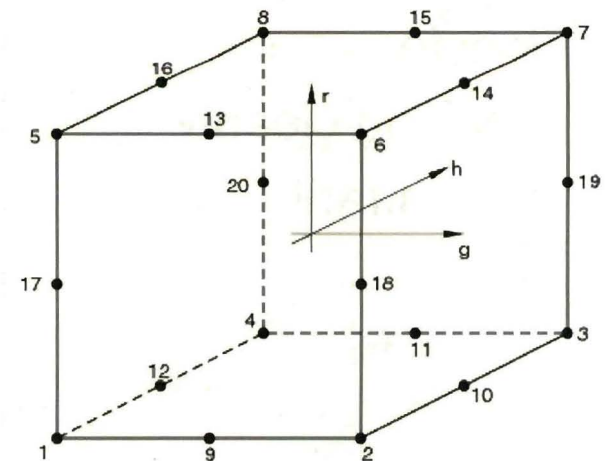
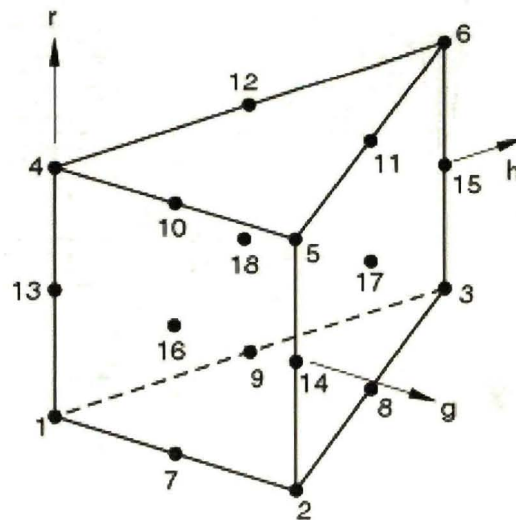
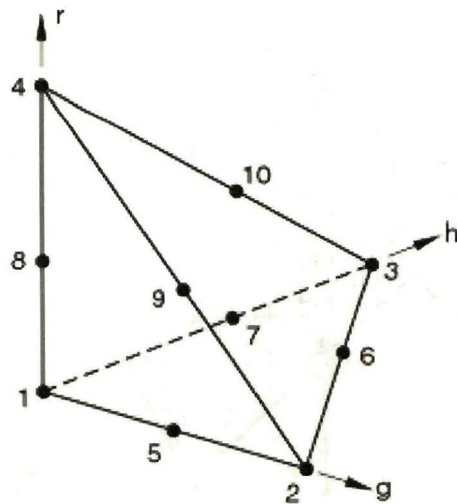


mesh universe / fill cell



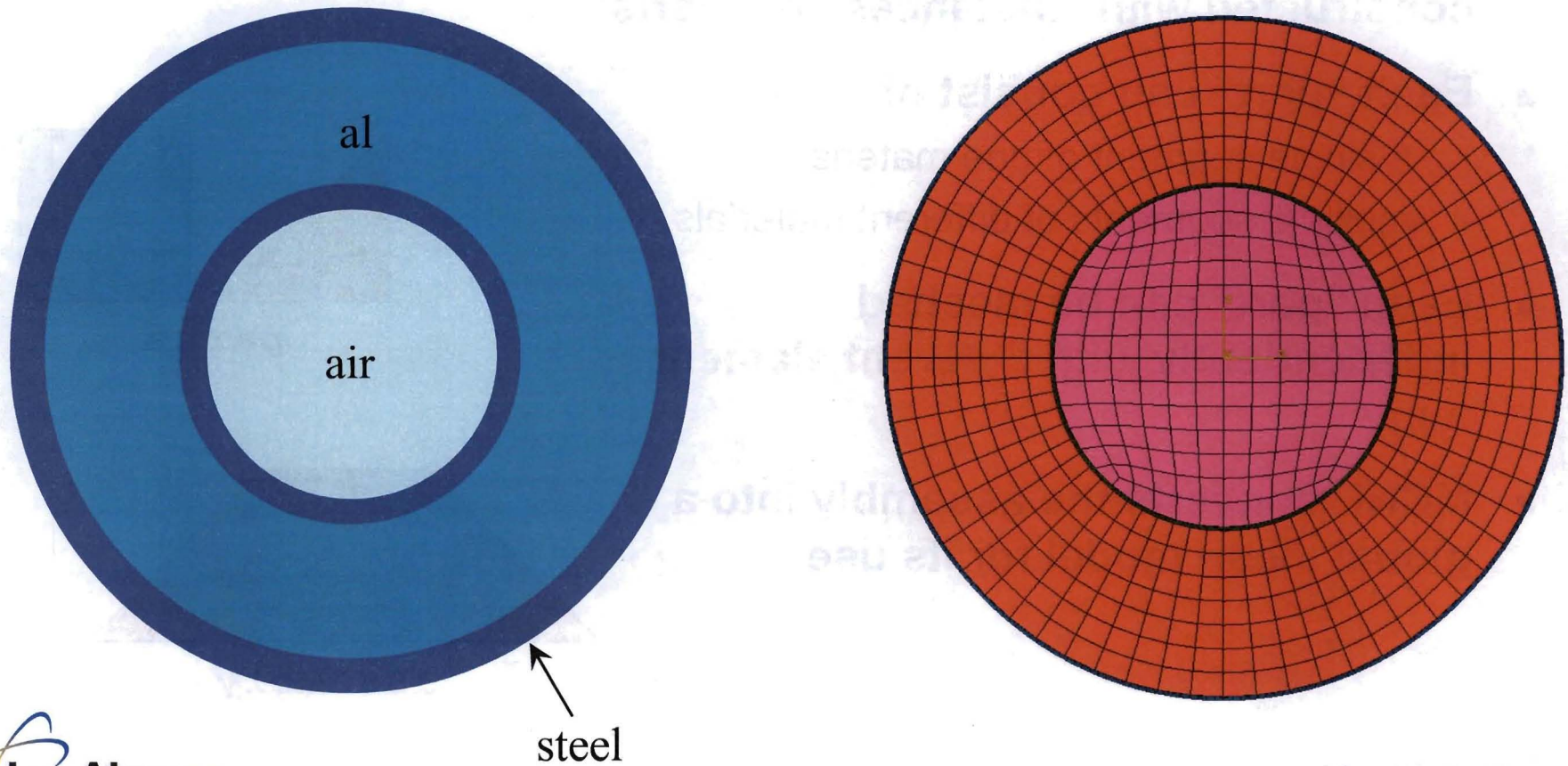
## Current Implementation

Unstructured mesh with 4-, 5-, and 6-sided elements generated by the ABAQUS<sup>®</sup> finite element program. **Surfaces may be bilinear.**



Currently, no mid-point nodes permitted.

## Osaka Aluminum Sphere Benchmark Problem

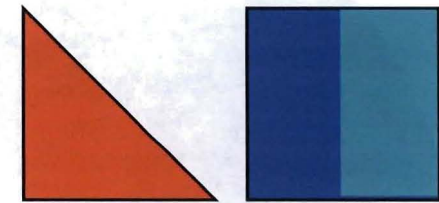


# Background: Constructing A Mesh Geometry

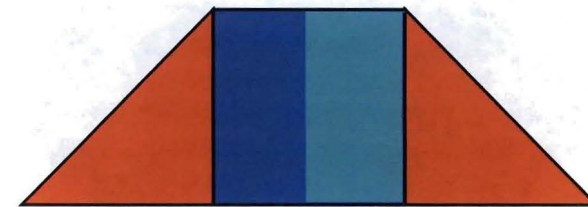
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## Created with ABAQUS

- The final model is an “assembly” constructed with “instances” of “parts”
- Each “part” can consist of
  - a single segment of one material
  - multiple segments of different materials
- Each “part” can be meshed independently **with different element types**
- MCNP converts the assembly into a global mesh model for its use



parts



assembly

## Background: Constructing A Mesh Geometry

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- How to map the material descriptions to the mesh?
- How to take advantage of MCNP's cell-base machinery?

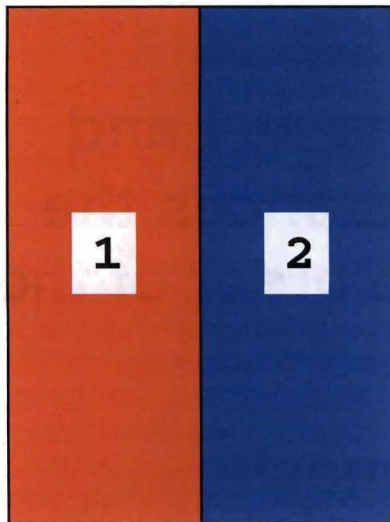
## Created with ABAQUS

- **Each part must contain three element sets (elsets) of data for:**
  - materials
  - statistics
    - Can not encompass multiple materials
    - Used for cell-based tallies
  - exterior surfaces

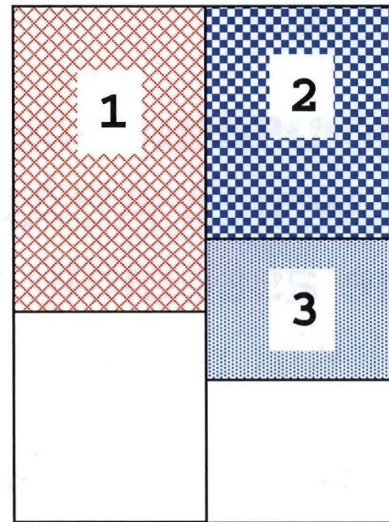
## Pseudo-Cells

- Each part--material--statistic region is automatically mapped to a pseudo-cell.
- The numbering of the pseudo-cells starts at 1 and occur in the order in which ABAQUS instances the parts to construct the assembly and the order of the statistical regions within the part.
- There must be one MCNP cell for each mesh pseudo-cell.
- The MCNP cell must contain the correct material definition for the pseudo-cell.

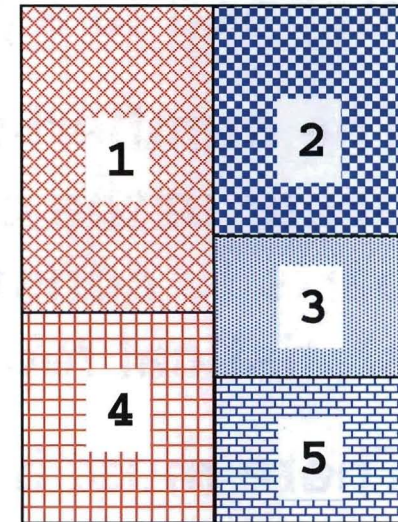
## Pseudo-Cell Example



1 part with 2 materials



3 defined & 2 undefined statistical regions



5 pseudo-cells  
(always consecutively numbered from 1)

## Contact Pair

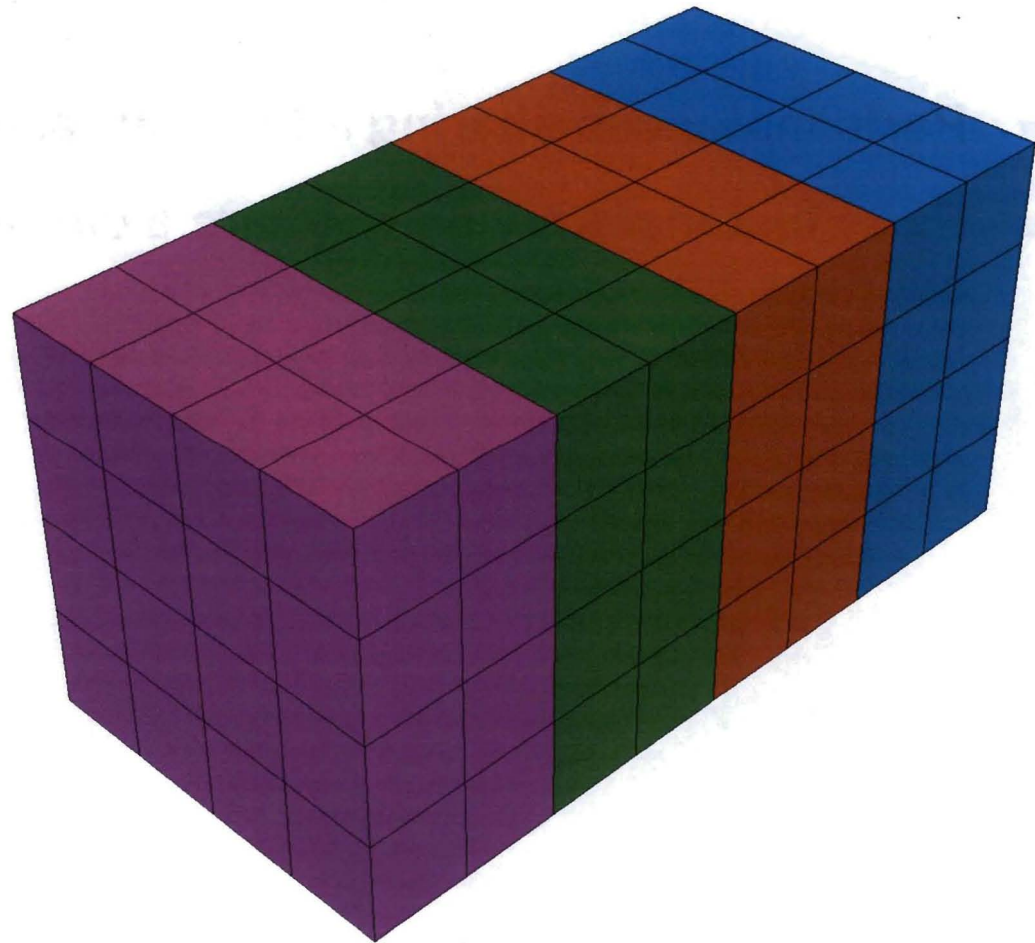
- Parts/Instances sharing a (flat) surface but not nodes.
- Parts/Instances trying to share a curved surface, resulting in overlaps and gaps.



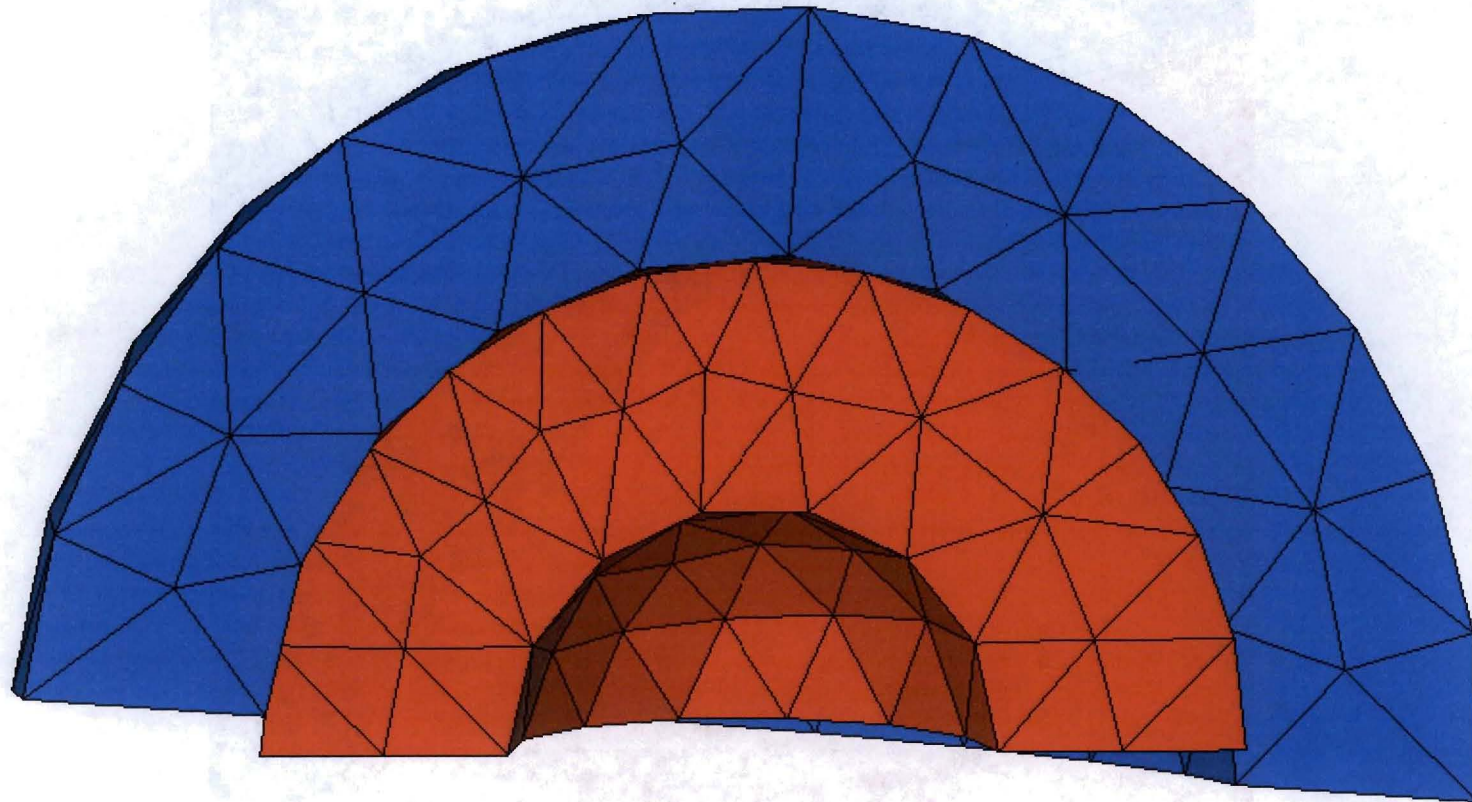
**4 Parts**

**3 Contact Pair  
Surfaces**

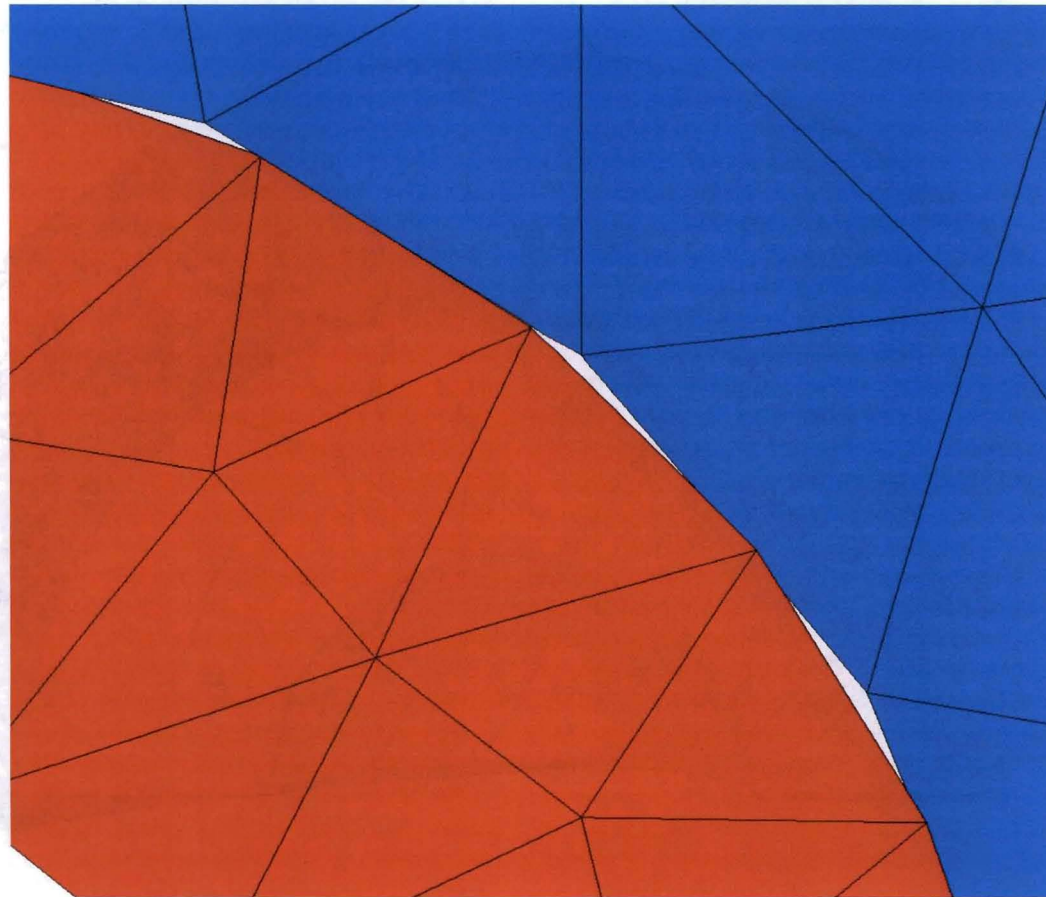
**Redundant nodes  
on each contact  
pair surface**



## Two Parts On A Curved Surface



## Gaps & Overlaps



### Modeling Considerations or “Style” Dictate Tracking Implementation

- **One part model with possibly many material sections**
  - quickest when tracking from element to element (use nearest neighbor search)
- **Multi-part model with contact pairs**
  - more work required to find the next element on the other side of the contact pair surface
- **Multi-part model with overlaps and gaps / re-entrant surfaces**
  - most work required; may need to look at all elements

User has control over the model

# Additional Requirements

- **Path length estimates of flux, energy deposition, and/or fission energy by mesh element**
  - Referred to as “elemental edits”
  - NO statistical uncertainties on results
  - Results output (including mesh geometry) in a special file
  - Dictates tracking implementation
    - Path length estimation (like MCNP) produces result in each mesh element through which the particle tracks.
    - Surface-to-surface “fast” tracking is not efficient in producing results in the mesh, but is desirable for transport speed up where edits aren’t needed.

# Current Restrictions

- Neutral particles only
- Charged particle tracking to be added later

# Input Cards

# Embedded Mesh Universe

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- Geometry mesh are embedded as the lowest level universe.
- Cell card requirements:
  - At least one cell card with an embedded universe parameter “embu” along with the “u” parameter is needed to embed the mesh universe.
  - “embu” value must coordinate with the number on the “embed” data card.
  - One cell card with “embu” parameter for each mesh “pseudo-cell”.
  - All cell cards with the “embu” parameter must appear first.
  - Immediately following the “embu” parameter cards must be a cell card with the “fill” parameter.

```
c      *** Cell cards ***
11     2  -7.8240  -8     embu=1  u=2
12     1  -1.2230  -8     embu=1  u=2
13     2  -7.8240  -8     embu=1  u=2
10     3  -0.0012  -8     embu=1  u=2
20     3  -0.0012  -8     fill=2
 6     3  -0.0012   8     -7
 7     0
```



## Embedded Mesh Control Card

<b>EMBEDn</b>	<b>meshgeo=</b>	<b>mgeoin=</b>	<b>meeout=</b>	<b>meein=</b>	<b>length=</b>
<b>n</b>					<b>embedded mesh universe number (only one card currently permitted)</b>
<b>meshgeo</b>					<b>mesh geometry type Current permitted values: abaqus</b>
<b>mgeoin</b>					<b>mesh input file name</b>
<b>meeout</b>					<b>elemental edits output file name</b>
<b>meein</b>					<b>elemental edits input file name (valid only in continuation runs)</b>
<b>length</b>					<b>conversion factor to centimeters for all mesh dimensions in input and output</b>

## Elemental Edits Control Card

**EMBEEn : <p1> embed= length= energy= time=**

**n** elemental edit number ending in 4, 6, or 7  
follows tally convention  
current maximum of 4 cards

**<p1>** particle designator from particle list  
current valid entrees: n or p

**embed** embedded mesh universe number  
must correspond to a valid embed card #

**energy** conversion factor from MeV/gm or jerks/gm  
for all energy related output

**time** conversion factor from shakes for all time  
related output

## Elemental Edit Energy Bins & Multipliers

**EMBEBn**    **B<sub>1</sub> B<sub>2</sub> ... B<sub>k</sub>**

**n**        elemental edit number; 0 is not valid.

**B<sub>i</sub>**        monotonically increasing upper energy of the *i*'*th* bin.

**EMBEMn**    **M<sub>1</sub> M<sub>2</sub> ... M<sub>k</sub>**

**n**        elemental edit number; 0 is not valid.

**M<sub>i</sub>**        monotonically increasing upper energy of the *i*'*th* bin.

## Elemental Edit Time Bins & Multipliers

**EMBTBn**     **B<sub>1</sub> B<sub>2</sub> ... B<sub>k</sub>**

**n**     elemental edit number; 0 is not valid.

**B<sub>i</sub>**     monotonically increasing upper time of the *i*'th bin.  
values in units of shakes (1 shake = 10<sup>-8</sup> s)

**EMBTMn**     **M<sub>1</sub> M<sub>2</sub> ... M<sub>k</sub>**

**n**     elemental edit number; 0 is not valid.

**M<sub>i</sub>**     monotonically increasing upper energy of the *i*'th bin.

# Concrete Cylinder Example

# Concrete Cylinder Geometry

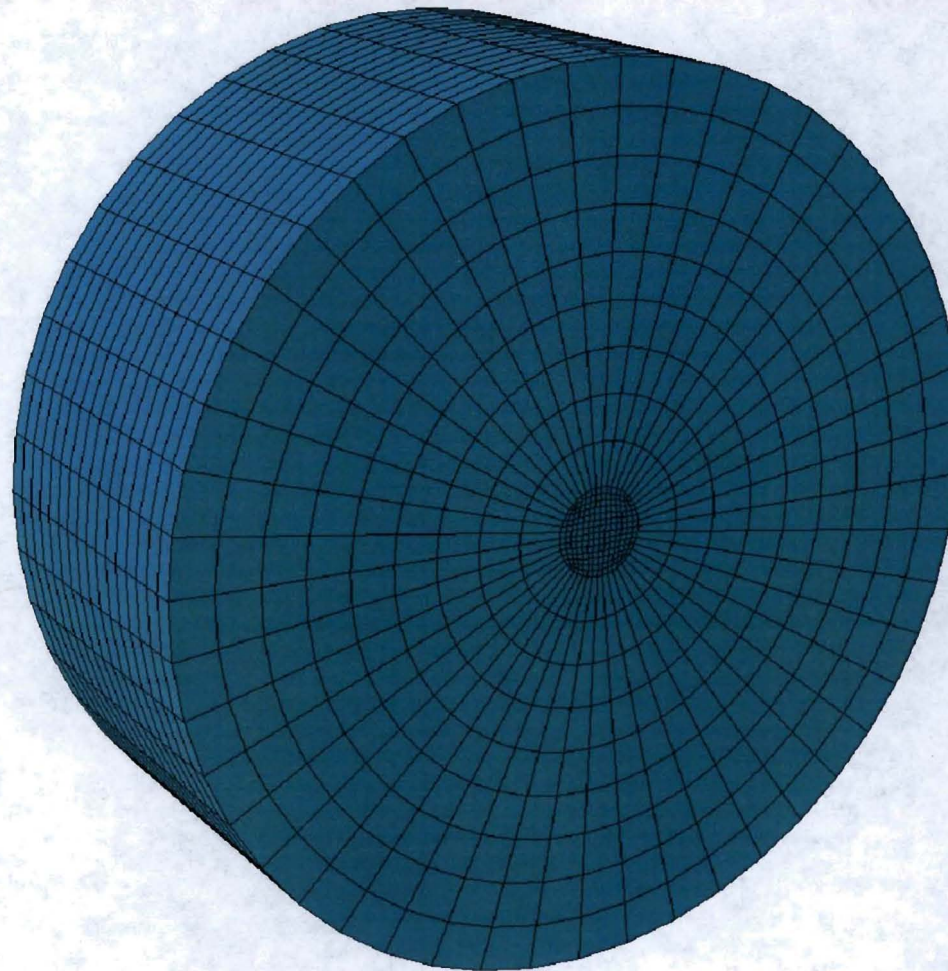
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$R = 100 \text{ cm}$

$H = 80 \text{ cm}$

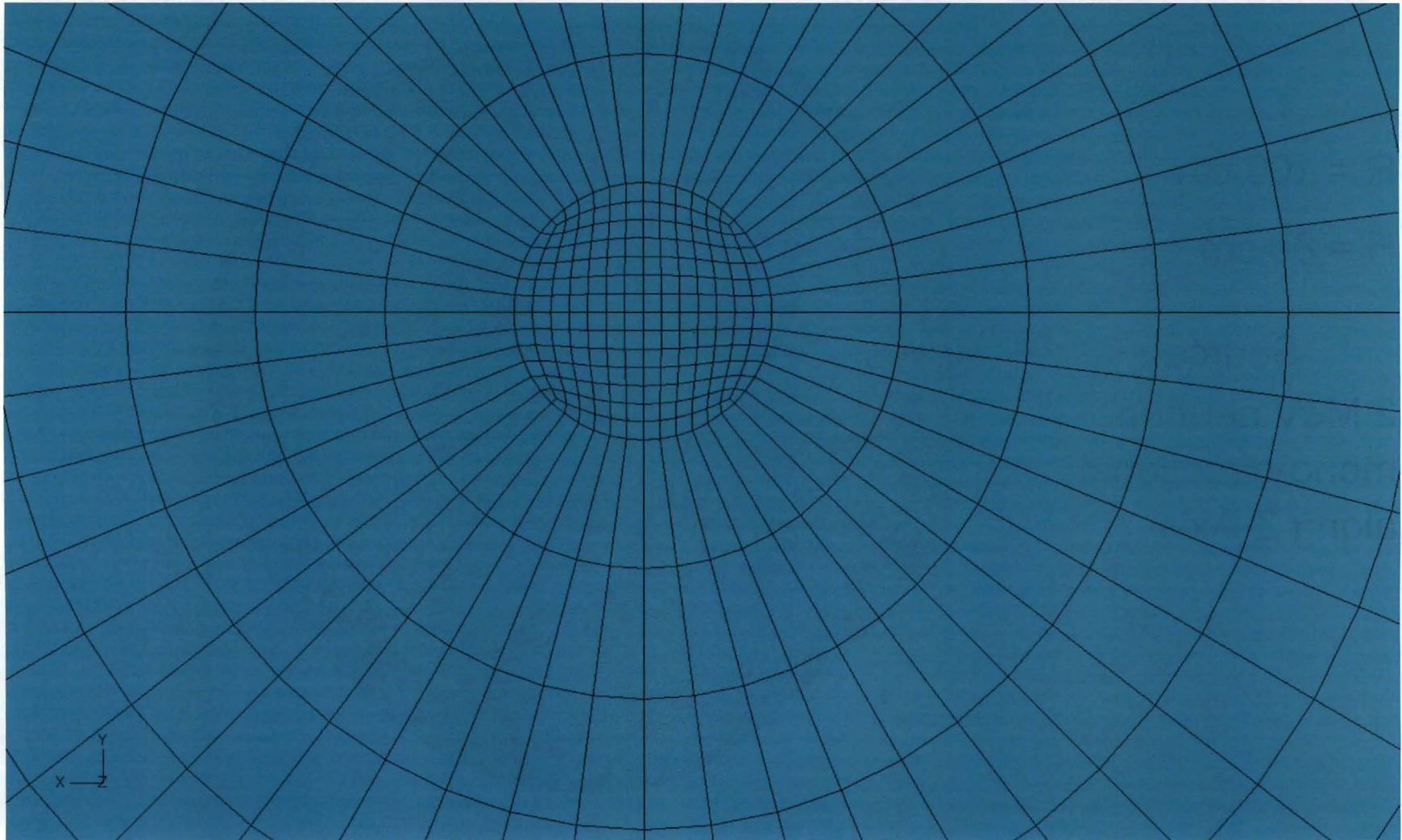
Source:

2 MeV neutrons;  
mono-directional  
along Z-Axis



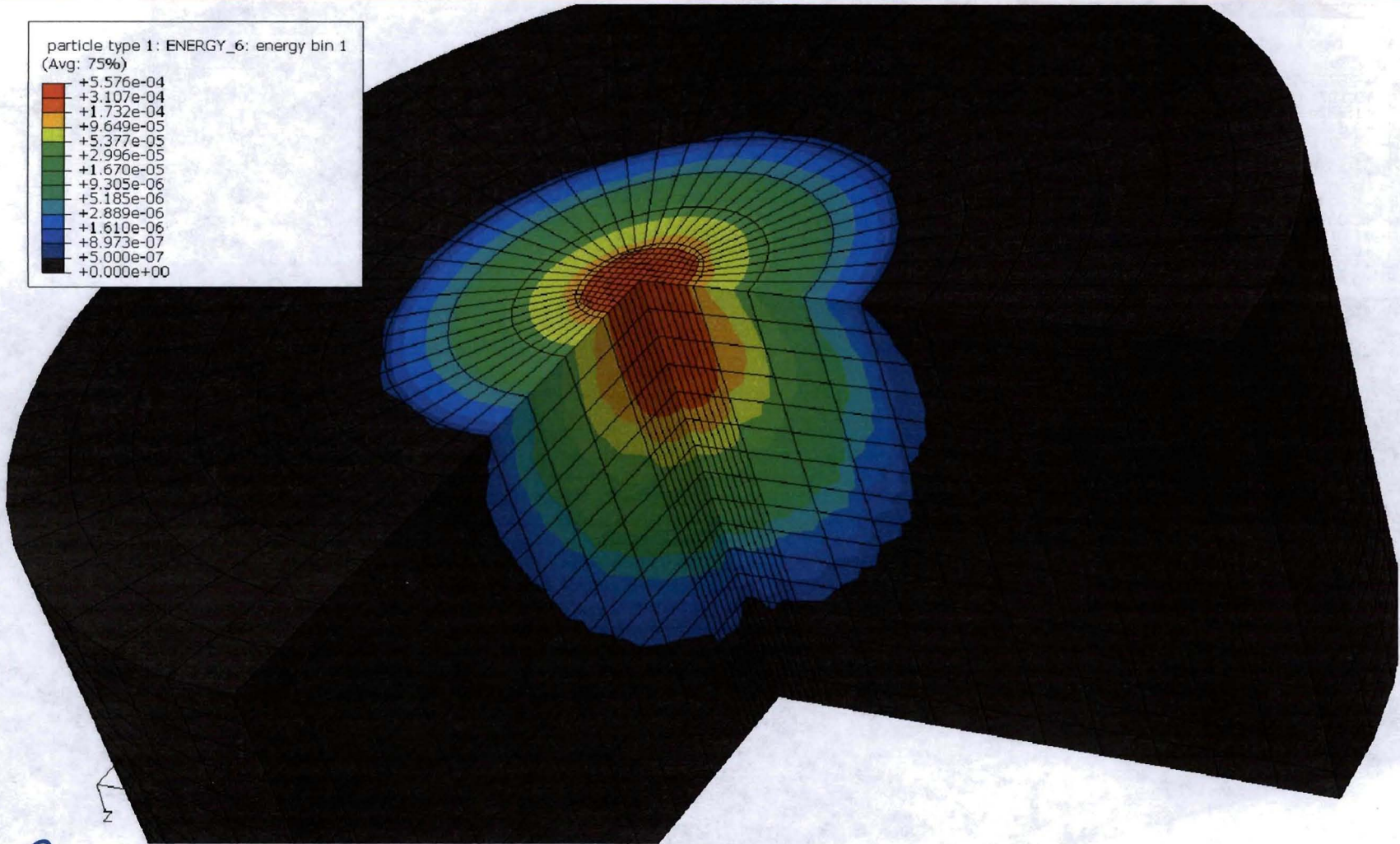
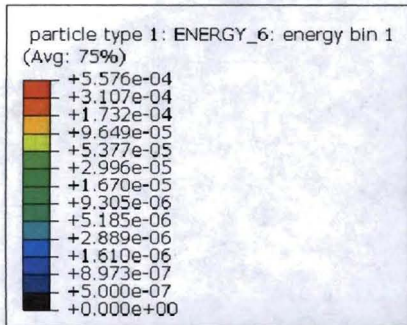
# Concrete Cylinder: 1 Part / 2 Mesh Zones

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# Total Energy Deposition: 3-D View With $\frac{3}{4}$ Geometry

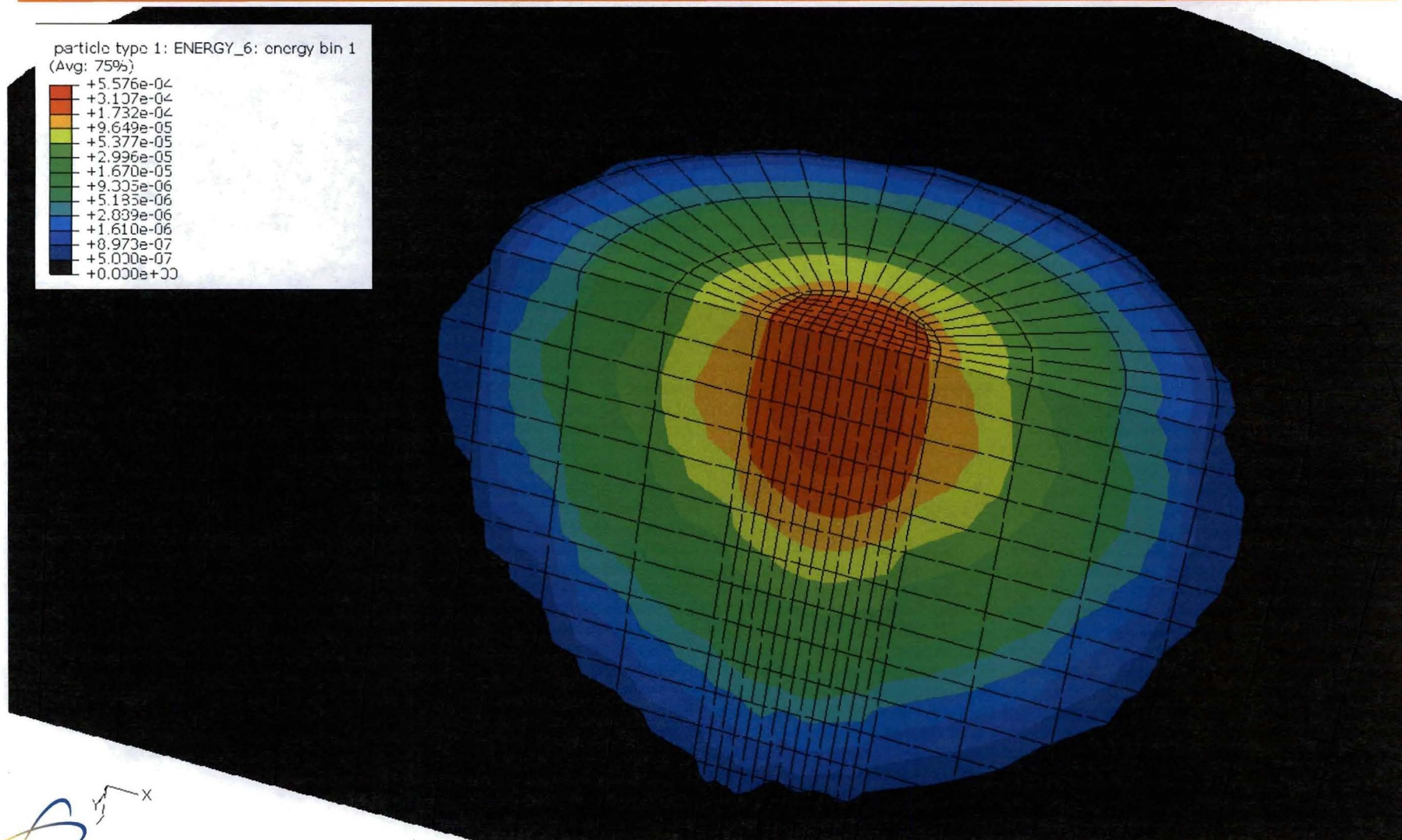
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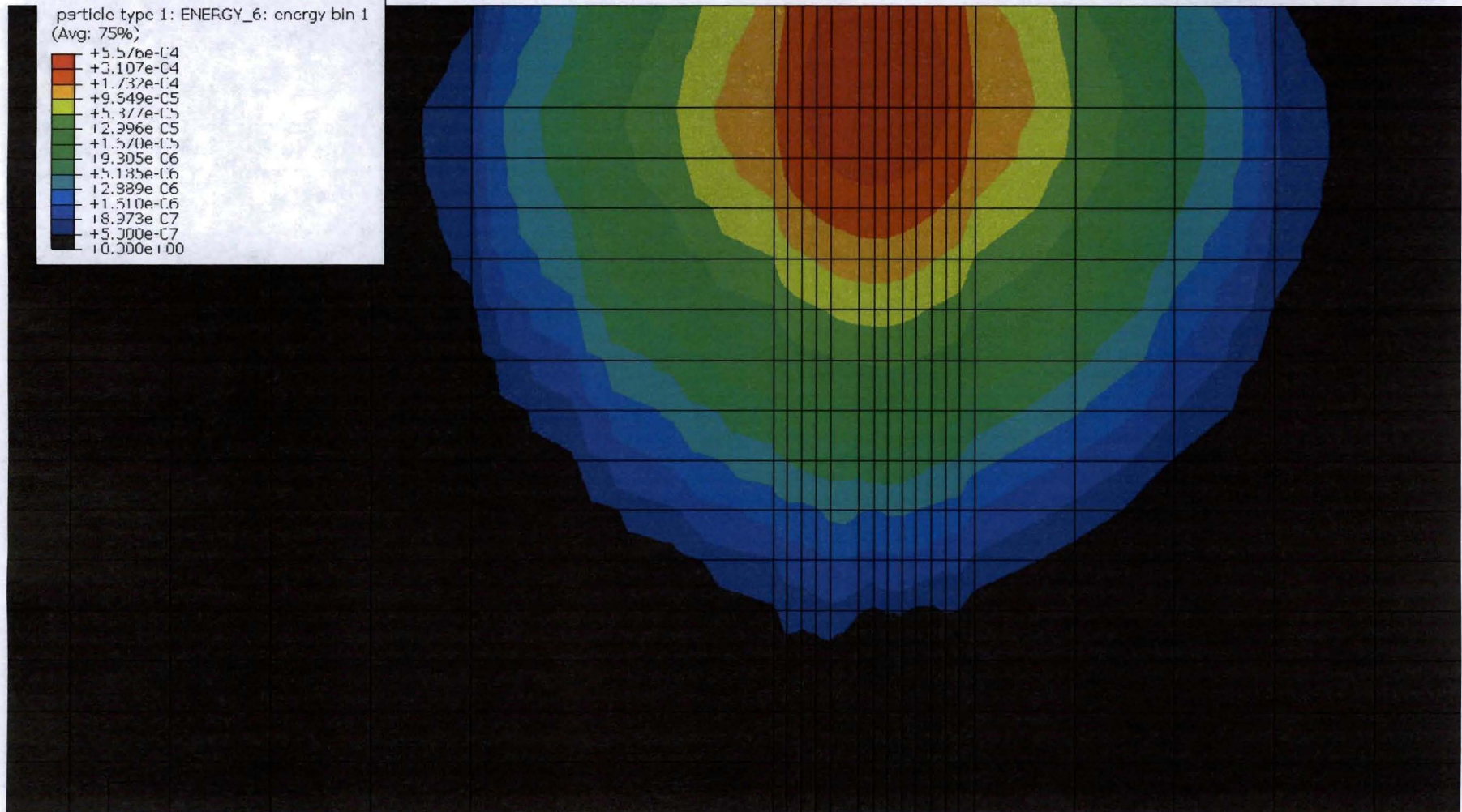
# Total Energy Deposition: 3-D View With Half Geometry

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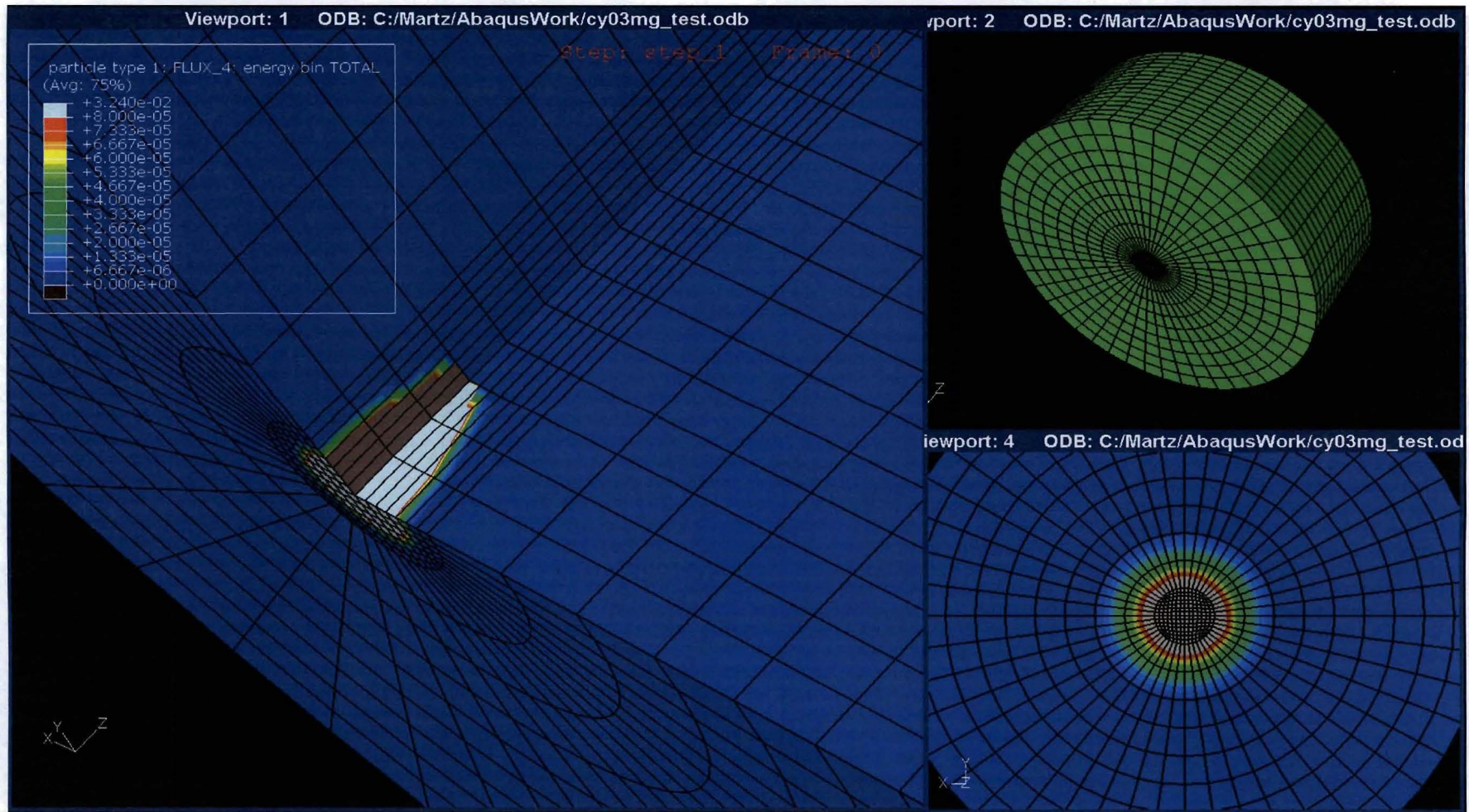
# Total Energy Deposition: 2-D View

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# Concrete Cylinder: total neutron flux movie

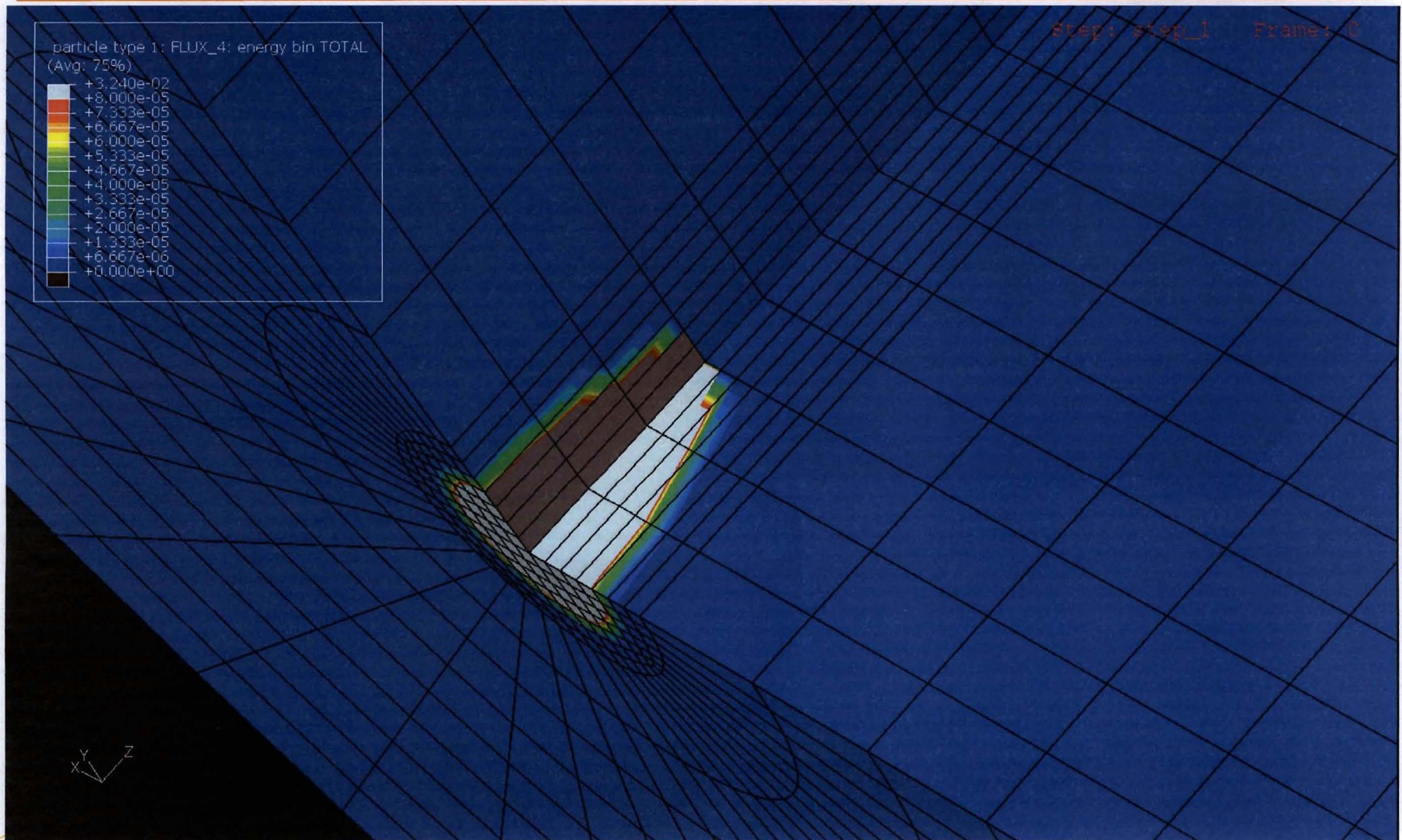
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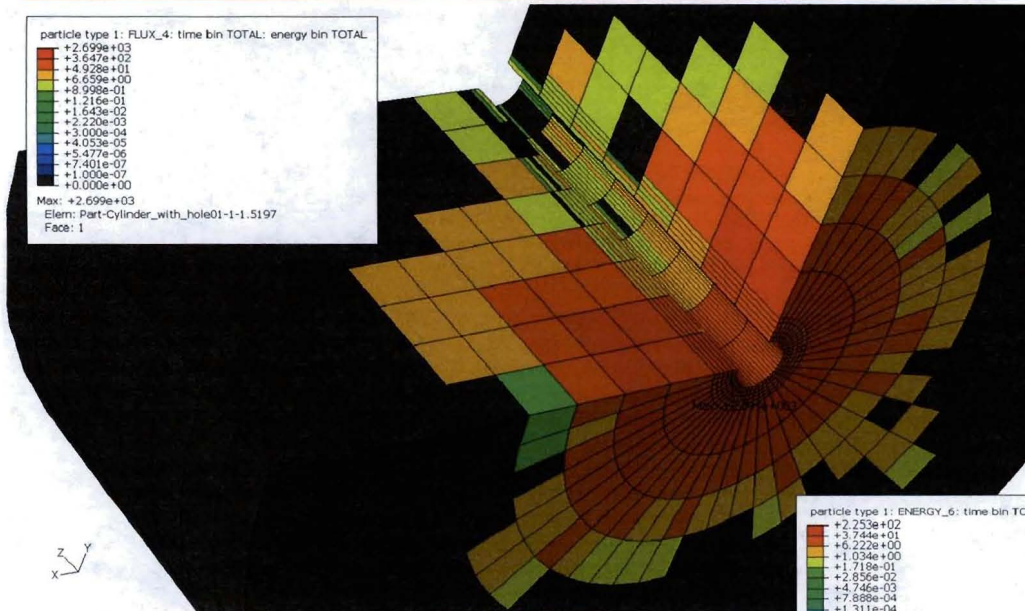
# Concrete Cylinder: total neutron flux movie

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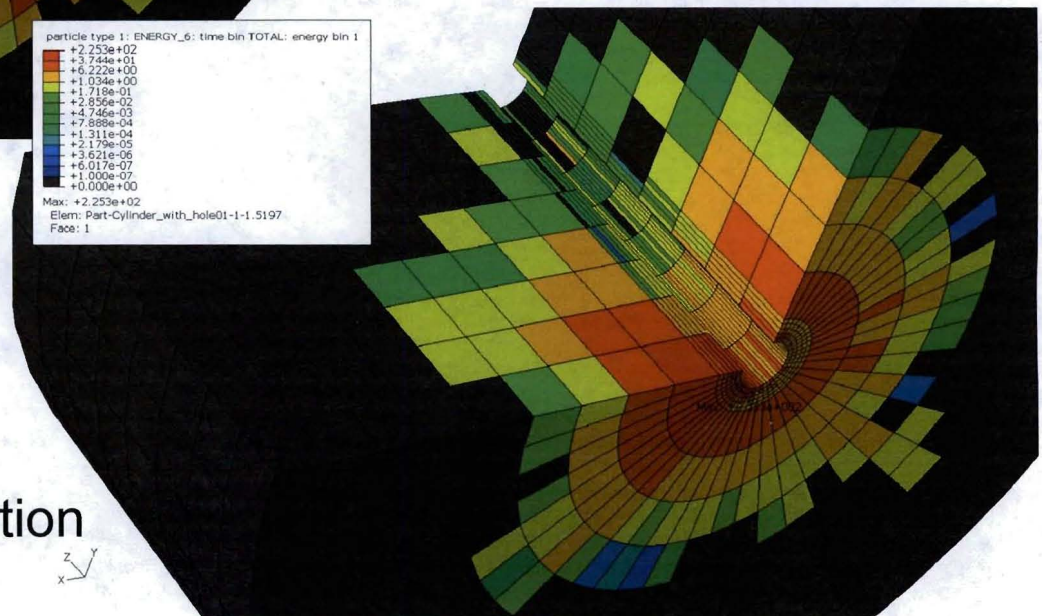
Step: step\_1 Frame: 0



# Concrete Cylinder With Penetration (5 cm radius) LA-UR-09-yyyy



Total neutron flux



Total neutron energy deposition

# Accomplishments

## Important Development Milestones To Date:

1. **Methods for “working with” unstructured mesh researched and programmed.**
  - Implemented with planar and bilinear surfaces for intersection & containment
  - skd-tree for searching mesh
  - Contact pairs, re-entrant particles, gaps / overlaps
2. **ABAQUS input parser**
3. **Mesh edit results file & visualization**
4. **Serial, omp, and mpi versions of the code are being tested.**
5. **Presented some preliminary results at the ANS RPSD08 meeting**

# Near Term Goals



### Next 6 Months

1. Continue testing & “bug” fixing
2. Resolve integration issues
3. Present user interface to MCNP Change Control Board
4. Implement surface-to-surface fast tracking
5. Implement quadratic surfaces
6. Technical society presentations
7. Documentation
8. Interest others in using these tools or fund further development